## Fukushima Daiichi Nuclear Power Station: Am and Cm analysis result in the soil

## 1. Analysis result

(Unit: Bq/kg·dry soil)

Sampling spot (): Distance from the stack of Unit 1, 2	Data sampling/Analys is organization	Pu-238 <sup>*1</sup>	Pu-239 <sup>*1</sup> Pu-240 <sup>*1</sup>	U-234 <sup>*2</sup>	U-235*2	U-238*2	Am-241	Cm-242	Cm-243 Cm-244
Playground (west-northwest approx. 500m)	May 16 Japan Chemical Analysis Center	(1.3±0.13) ×10 <sup>-1</sup>	$(6.5 \pm 0.86)$ × $10^{-2}$	$(1.4 \pm 0.08)$ × $10^{1}$	$(5.2 \pm 1.0)$ $\times 10^{-1}$	$(1.5 \pm 0.09)$ × $10^{1}$	N.D.	$(1.5 \pm 0.08)$ × $10^{0}$	$(9.0 \pm 1.3)$ × $10^{-2}$
Near the industrial waste disposal plant (south-southwest approx.		(3.8±0.60) ×10 <sup>-2</sup>	$(1.8 \pm 0.41)$ × $10^{-2}$	$(7.5 \pm 0.48)$ × $10^{0}$	$(5.4 \pm 1.1)$ $\times 10^{-1}$	$(7.0 \pm 0.45)$ × $10^{\circ}$	N.D.	(7.2±0.38) ×10 <sup>-1</sup>	(2.2±0.51) ×10 <sup>-2</sup>
Playground (west-northwest approx. 500m)	- May 23 Japan Chemical Analysis Center	$(1.5 \pm 0.13)$ × $10^{-1}$	$(5.9 \pm 0.77)$ × $10^{-2}$	$(1.4 \pm 0.06)$ × $10^{1}$	$(7.5 \pm 1.0)$ × $10^{-1}$	$(1.4 \pm 0.06)$ × $10^1$	$(1.6 \pm 0.47)$ × $10^{-2}$	$(1.7 \pm 0.07)$ × $10^0$	$(1.0 \pm 0.12)$ × $10^{-1}$
Near the industrial waste disposal plant (south-southwest approx.		(1.0±0.11) ×10 <sup>-1</sup>	(4.1±0.64) ×10 <sup>-2</sup>	$(4.8 \pm 0.34)$ × $10^{0}$	$(3.3 \pm 0.82)$ × $10^{-1}$	(5.3±0.37) ×10°	(2.1±0.49) ×10 <sup>-2</sup>	(1.3±0.06) ×10°	(9.3±1.1) ×10 <sup>-2</sup>
Average nuclide density ratio of fuel in Units 1 to 3 (ratio in case the ratio of Pu-238 is considered as 1)*3		1	-	-	-	-	0 . 1	1 0	1

<sup>\*1:</sup> Released on June 4, 2011 \*2: Released on June 25th, 2011 \*3: Values calculated by ORIGEN Code (round number)

## 2.Evaluation

Detected Am and Cm are considered to derive from the accident due to following reasons.

- Cm-242, Cm-243 and Cm-244 are nuclides that do not exist in the natural world. In particular, Cm-242 whose half-life is relatively short (approximately 160 days) was detected.
- The density ratio of each nuclides (Am-241/Cm-242/Cm-243,Cm-244) to Pu-238 in the sample , , and is almost the same as the average nuclide density ratio of fuel in Units 1 to 3.

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Pu-238 in the sample : (Am-241/Cm-242/Cm-243,Cm-244) 1: (-/12/0.7) Pu-238 in the sample : (Am-241/Cm-242/Cm-243,Cm-244) 1: (-/19/0.6) Pu-238 in the sample : (Am-241/Cm-242/Cm-243,Cm-244) 1: (0.1/11/0.7) Pu-238 in the sample : (Am-241/Cm-242/Cm-243,Cm-244) 1: (0.2/13/0.9)
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END