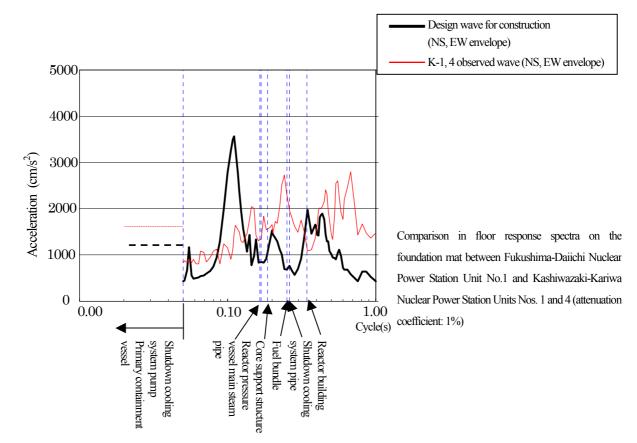
## A summary of the "Report on the results of a general study of potential seismic impact on the main facilities of the nuclear power stations made on the basis of the observational data at the Kashiwazaki-Kariwa Nuclear Power Station"

## 1 Content of study

The floor response spectra on the foundation mat of the reactor building observed at the Kashiwazaki-Kariwa Nuclear Power Station and those from the design-basis earthquake motions for the Fukushima-Daiichi and Fukushima-Daini Nuclear Power Stations were compared to investigate impact on functional maintenance of the main facilities of the nuclear power stations of our company.



## 2 Results of study

It is believed that safety functions of the main facilities which play an important function for safety of the individual units of the Fukushima-Daiichi and Fukushima-Daini Nuclear Power Stations can be maintained to withstand such earthquake motions as were observed in the Kashiwazaki-Kariwa Nuclear Power Station during the Niigata-Chuetsu-oki Earthquake in 2007.

Results of a general study on seismic influences (an example of the Fukushima-Daiichi Nuclear Power Station Unit No.
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Object facilities	Spectral ratio: $\alpha^{*1}$	Capacity margin of facilities: $\beta^{*2}$	Judg- ment
Reactor pressure vessel	1.59	2.30	0
Core support structure	1.55	2.24	0
Shutdown cooling system pump	1.94	3.55	0
Shutdown cooling system pipe	2.57	2.77	0
Main steam system pipe	1.59	3.28	0

\*1 represents the "response acceleration spectrum from the earthquake motions observed at the Kashiwazaki-Kariwa Nuclear Power Station / response acceleration spectrum from the design earthquake motions for the Fukushima-Daiichi Nuclear Power Station" in the proper period of the object facilities. When  $\alpha$  is 1 or below, it means that the floor response spectrum from the design earthquake motions for the Fukushima-Daiichi Nuclear Power Station exceeds that for the Kashiwazaki-Kariwa Nuclear Power Station, and thus, safety functions are considered to be maintainable.

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\*2 indicates the "permitted value / response value" of the design

Primary containment vessel	1.94	4.64	0
Reactor building	1 or less	_	0
Control rod (insertability)		<sup>*3</sup> ≤ 40mm e Permitted value	0

earthquake motions for the object facilities. Even in the event that  $\alpha$  exceeds 1, but so far as  $\alpha \leq \beta$  is maintained, safety functions can be maintained since the margin to the permitted value of the object facilities exceeds the ratio of the floor response spectrum of the Kashiwazaki-Kariwa Nuclear Power Station to the floor response spectrum of the Kashiwazaki-Kariwa Nuclear Power Station.

\*3 With respect to the Fukushima-Daiichi Nuclear Power Station Units Nos. 1 through 5, an analysis was made using the fuel bundle analytical model concerning control rod insertability based on the earthquake motions observed at the Kashiwazaki-Kariwa Nuclear Power Station to identify that it is below the permitted value (40mm).

It was also confirmed for units other than the Fukushima-Daiichi Nuclear Power Station Unit No.1 that safety functions can be similarly maintained with respect to major safety-significant facilities.