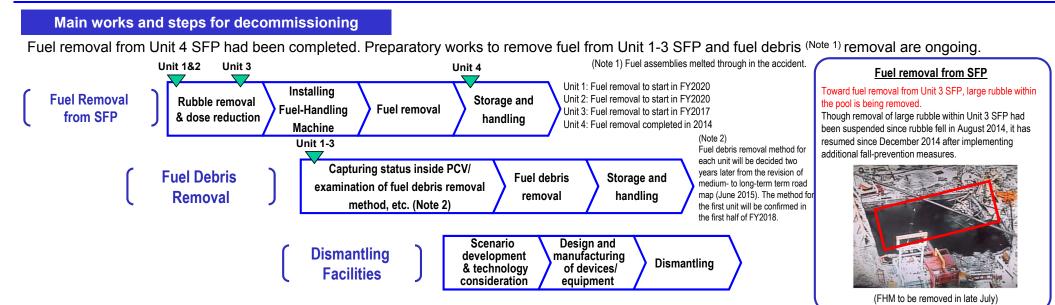
Summary of Decommissioning and Contaminated Water Management

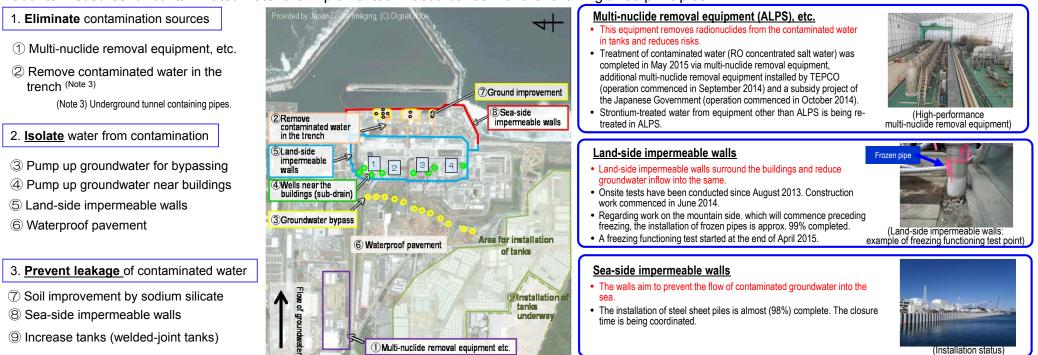
Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment

June 25, 2015



Three principles behind contaminated water countermeasures

Water to cool fuel having melted in the accident is mixed with groundwater and approx. 300 tons of contaminated water is generated daily. Countermeasures for contaminated water are implemented in accordance with the following three principles:



1/10

Summary of Decommissioning and Contaminated Water Management June 25, 2015 Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment

Progress status

◆ The temperatures of the Reactor Pressure Vessel (RPV) and the Primary Containment Vessel (PCV) of Units 1-3 have been maintained within the range of approx. 20-45°C^{*1} for the past month. There was no significant change in the density of radioactive materials newly released from Reactor Buildings in the air¹². It was evaluated that the comprehensive cold shutdown condition had been maintained. The values vary somewhat depending on the unit and location of the thermometer.

*2 In May 2015, the radiation exposure dose due to the release of radioactive materials from the Reactor Buildings was evaluated as less than 0.0016 mSv/year at the site boundaries This is approx. 1/70 of the annual radiation dose by natural radiation (annual average in Japan; approx. 2.1 mSv/year)

Status of dismantling of Unit 1 building cover

On May 21, a disconnect was found with the balloon installed on the Reactor Building 3rd floor equipment hatch. The investigation confirmed rubble and antiscattering agents had accumulated on the rain cover over the balloon.

As the release amount was sufficiently low, regardless of a balloon, it will not be recovered. However,

countermeasures will be taken to reduce wind inflow with the aim of controlling the release.

The building cover is being removed with anti-scattering measures steadily implemented and safety prioritized above all.

Resumption of rubble removal from the Unit 3 spent fuel pool

For large rubble removal from the Unit 3 spent fuel pool (SFP), which has been suspended due to the failure of the camera mounted on the crane, the camera was replaced during the annual crane inspection.

From June 22, the removal of large rubble from Unit 3 SFP resumed.

The fuel handling machine, which is the largest rubble within the pool, will be removed carefully using a dedicated lifting tool.

Status of the freezing functioning test of land-side impermeable walls

From April 30, the freezing functioning test started at 18 points (58 frozen pipes, approx. 6% on the mountain side). This test is to confirm that there is no problem with the facility-wide operation status and that the earth temperature has decreased.

As a continuous change in the groundwater level was identified at a freezing functioning test point, coolant supply to this point has been suspended since June 3.

Revision of the Mid-and-

Long-Term Roadmap On June 12, at the Inter-Ministerial Council for Contaminated Water and Decommissioning Issues, the Mid-and-Cong-Term Roadmap was resolved. On June 15, at the Fukushima Advisory Board on Decommissioning and Contaminated Water Management, the revised Mid-and-Long Term Roadmap was explained. The decommissioning and contaminated water countermeasures will steadily proceed based on continued close communication with local citizens and their requests.

Building cover Blowout panel Cover for fuel removal (closed) Removed fuel (assemblies) Reactor Building (R/B) **1533**/1533 Freezing Primarv Fuel removal completed on December 22, 2014) unctioning Containmen Vessel test started Land-side impermeable walls (PCV) 392 615 566 Water Wate Reactor Pressur Vessel (RPV) Fuel deb Vent pipe 1384<1205>/1551 orus rooi

Operation start of large rest house

At the large rest house with a capacity of approx. 1,200 workers, operation started on May 31 and meal services at the dining room commenced on June 1. This rest house is used for workers to rest, conduct office work and receive safety checks before starting work.

Based on the judgement that renovation would be required to further improve it from a hygiene perspective the dining room has been temporarily closed since June 9. Following the renovation, operations will resume from late July. During the suspension period, service hours of the dining room at the new Administration Office Building have been extended to improve usability for workers.

Start of drilling for solid waste storage

To store rubble safely, additional solid waste storage (9th) with a capacity of approx. 110,000 of 200 litter drums will be installed. Following the pre-construction work, including site preparation, drilling commenced on June 8.

Acceptance of rubble started at the 3rd temporary soil cover type storage commenced

From June 23, acceptance of rubble started at the 3rd temporary soil cover type storage, which was

installed to store waste appropriately. This facility stores metal and concrete rubble at a surface dose rate of 30 mSv/h or less.

<First rubble acceptance>

Leakage from the transfer hose to the Turbine Building

On May 29, leakage from a pressure hose was detected during the transfer from a 1,000t notch tank to the Turbine Building.

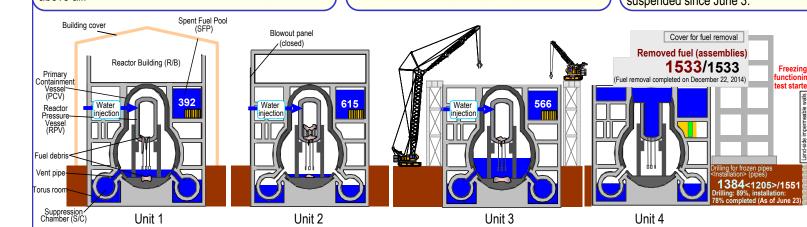
The leaking water flowed into the port via drainage channel K. However, the water was collected from drainage channel K, etc. and judging from no significant variation in radiation density at the port entrance and in the outer sea, it was considered that there was no impact outside the port.

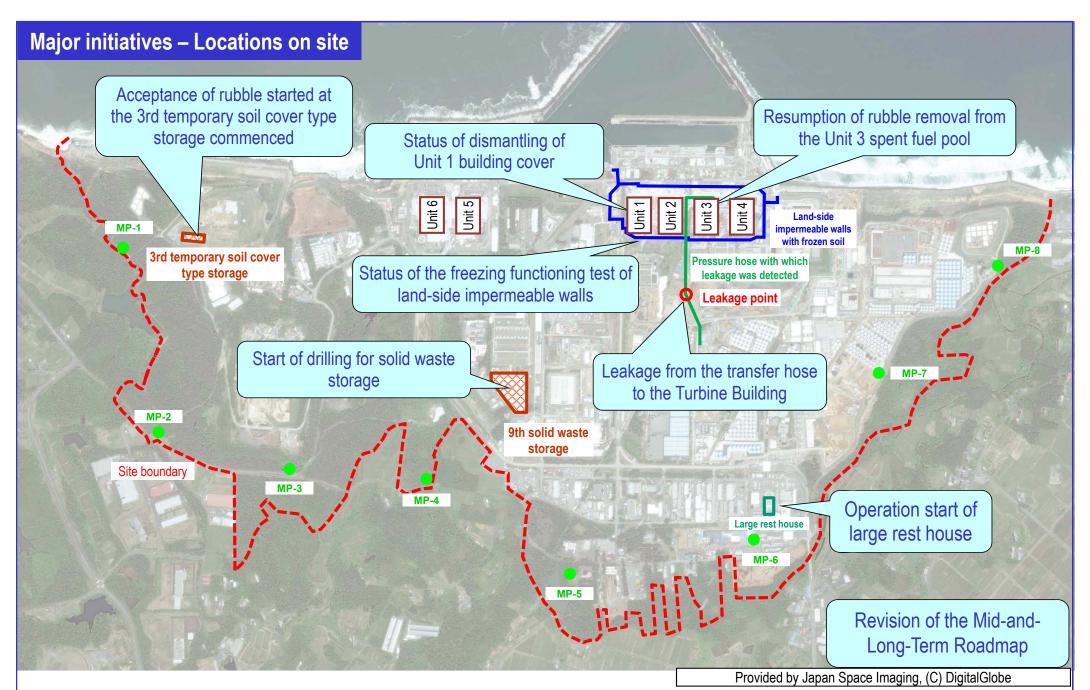
As a countermeasure, the relevant pressure hose was replaced with a polyethylene pipe.



<Leakage point>







Data of Monitoring Posts (MP1-MP8.)

Data of Monitoring Posts (MPs) measuring airborne radiation rate around site boundaries show 0.967 - 3.948 µSv/h (May 27 – June 23, 2015).

We improved the measurement conditions of monitoring posts 2 to 8 for precise measurement of air dose rate. Construction works such as tree-clearing, surface soil removal and shield wall setting were implemented from Feb. 10 to Apr. 18, 2012.

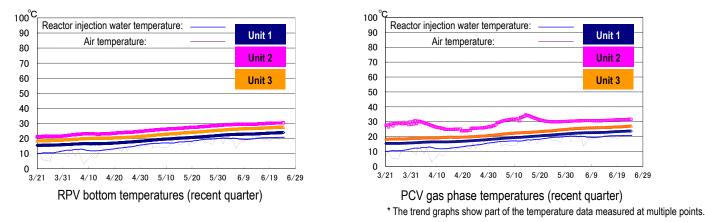
Therefore monitoring results at these points are lower than elsewhere in the power plant site.

The radiation shielding panel around monitoring post No. 6, which is one of the instruments used to measure the radiation dose of the power station site boundary, were taken off from July 10-11, 2013, since the surrounding radiation dose has largely fallen down due to further cutting down of the forests etc.

I. Confirmation of the reactor conditions

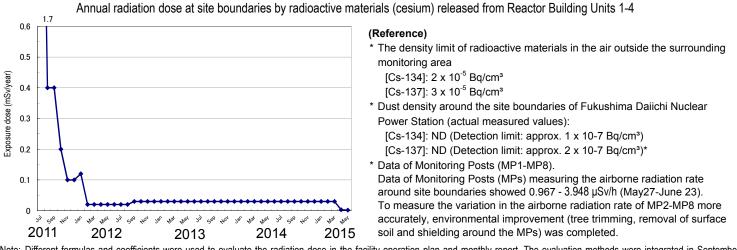
1. Temperatures inside the reactors

Through continuous reactor cooling by water injection, the temperatures of the Reactor Pressure Vessel (RPV) bottom and the Primary Containment Vessel (PCV) gas phase have been maintained within the range of approx. 20 to 45°C for the past month, though they vary depending on the unit and location of the thermometer.



2. Release of radioactive materials from the Reactor Buildings

As of May 2015, the density of radioactive materials newly released from Reactor Building Units 1-4 in the air and measured at the site boundaries was evaluated at approx. 4.5×10⁻¹¹ Bg/cm³ for Cs-134 and 1.2×10⁻¹⁰ Bg/cm³ for Cs-137 respectively. The radiation exposure dose due to the release of radioactive materials was less than 0.0016 mSv/year at the site boundaries.



Note: Different formulas and coefficients were used to evaluate the radiation dose in the facility operation plan and monthly report. The evaluation methods were integrated in September 2012. As the fuel removal from the spent fuel pool (SFP) commenced for Unit 4, the radiation exposure dose from Unit 4 was added to the items subject to evaluation since November 2013

The evaluation has been changed to a method considering the values of continuous dust monitors since FY2015, with data to be evaluated monthly and announced the following month.

3. Other indices

There was no significant change in indices, including the pressure in the PCV and the PCV radioactivity density (Xe-135) for monitoring criticality, nor was any abnormality of cold shutdown condition or sign of criticality detected.

Based on the above, it was confirmed that the comprehensive cold shutdown condition had been maintained and the reactors remained in a stabilized condition.

II. Progress status by each plan

1. Contaminated water countermeasures

To tackle the increase in accumulated water due to groundwater inflow, fundamental measures to prevent such inflow into the Reactor Buildings will be implemented, while improving the decontamination capability of water treatment and preparing facilities to control the contaminated water

- Operation of groundwater bypass \geq
- From April 9, 2014, the operation of 12 groundwater bypass pumping wells commenced sequentially to pump up Liaison Office for the Decommissioning and Contaminated Water Issue of the Cabinet Office. As of June 24, 2015, met operational targets.
- It was confirmed that the groundwater inflow into the buildings had decreased by approx. 80 m³/day, based on the evaluation data to date, through measures such as groundwater bypass and water stoppage of the High Temperature Incinerator Building (HTI) (see Figure 1).
- It was confirmed that the groundwater level at the observation holes had decreased by approx. 5-20 cm compared to the level before pumping at the groundwater bypass started.
- No. 12: from May 25).

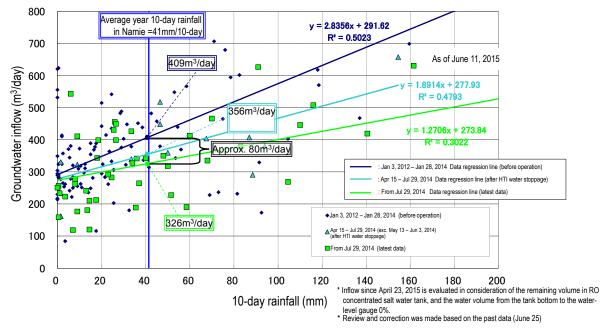


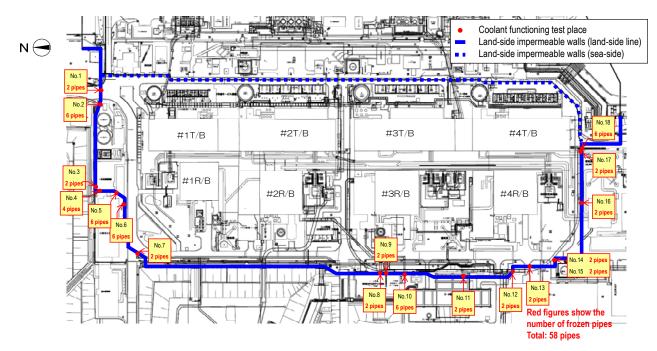
Figure 1: Analytical results of inflow into buildings

- Construction status of land-side impermeable walls \geq
- To facilitate the installation of land-side impermeable walls surrounding Units 1-4 (a subsidy project of the Ministry of Economy, Trade and Industry), drilling to place frozen pipes commenced (from June 2, 2014). Regarding the mountain side which will be frozen first, as of June 23, 2015 drilling at 1,249 points (approx. 99%, for frozen pipes: 1,025 of 1,036 points, for temperature-measurement pipes: 224 of 228 points) and installation of frozen pipes at 1,025 of 1,036 points (approx. 99%) had been completed (see Figure 3). The remaining construction will proceed after the necessary procedures have been performed.
- From April 30, the freezing functioning test is underway at 18 points (58 frozen pipes, approx. 6% on the mountain multiple observation wells, which had not been affected by freezing, exceeded the standard value for four consecutive days, supply of brine to this point was suspended since June 3.

groundwater. The release commenced from May 21, 2014 in the presence of officials from the Intergovernmental 111,583m³ of groundwater had been released. The pumped up groundwater has been temporarily stored in tanks and released after TEPCO and a third-party organization (Japan Chemical Analysis Center) confirmed that its quality

Due to a decrease in the flow rate of pumping well Nos. 5, 7, 8, 10 and 12, water pumping was suspended for cleaning (No. 5: from May 22, No. 7: from June 10, No. 8: from May 22 to June 17, No. 10: from April 27 to June 9,

side). The test confirmed that the facilities were operating correctly and the earth temperature has decreased. As the change in groundwater level at the observation well near the freezing functioning test point No. 7 from the level of



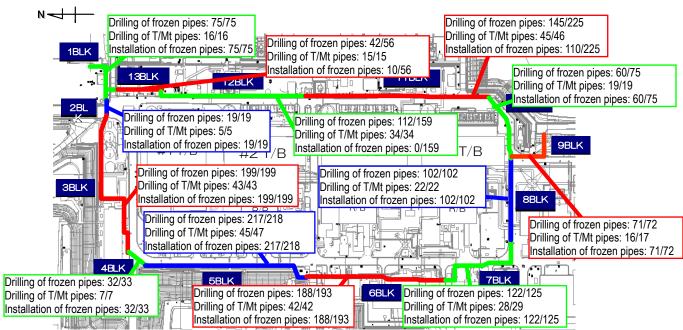


Figure 3: Drilling status for frozen-soil impermeable walls and installation of frozen pipes

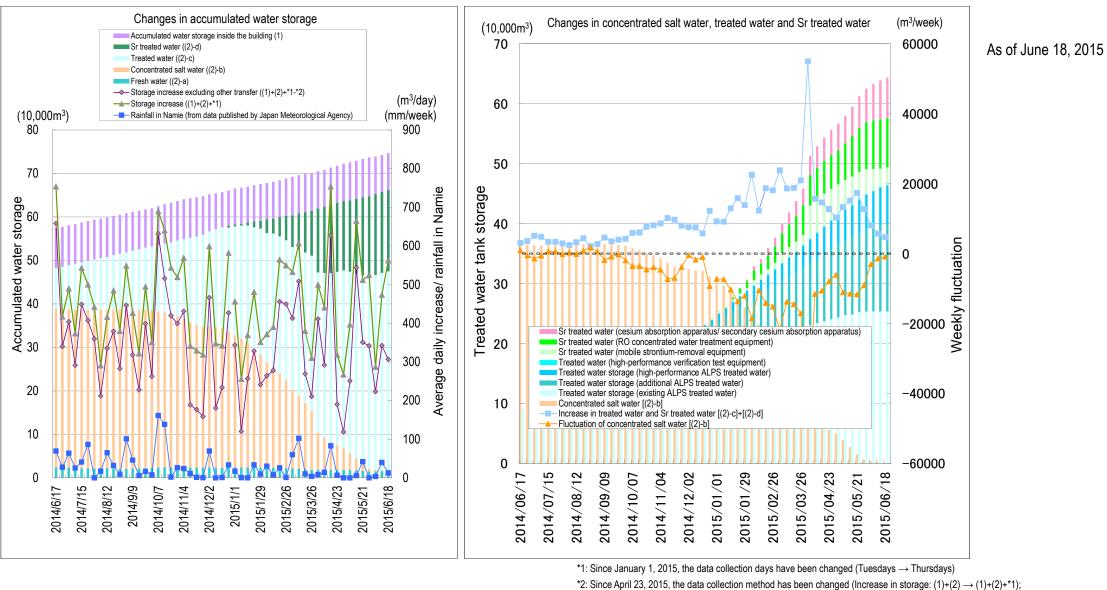


Figure 4: Status of accumulated water storage

increase in storage excluding other amounts transferred: $(1)+(2)-*2 \rightarrow (1)+(2)+*1-*2)$,

*3: Water amount with which water-level gauge indicates 0% or more

*4: Review and correction was made based on past data

Figure 2: Freezing functioning test place on land-side impermeable walls

Operation of multi-nuclide removal equipment

- Regarding multi-nuclide removal equipment (existing, additional and high-performance), hot tests using radioactive water are underway (for existing equipment, System A: from March 30, 2013, System B: from June 13, 2013, System C: from September 27, 2013; for additional equipment, System A: from September 17, 2014, System B: from September 27, 2014, System C: from October 9, 2014; for high-performance equipment, from October 18, 2014).
- As of June 18, the volumes treated by existing, additional and high-performance multi-nuclide removal equipment were approx. 254,000, 146,000 and 64,000 m³ respectively (including approx. 9,500 m³ stored in the J1(D) tank, which contained water with a high density of radioactive materials at the System B outlet).
- For Systems A and C of existing multi-nuclide removal equipment, facility inspections are underway and additional absorption vessels are being installed to improve their performance (from May 24). System B will be operated as necessary to treat wastewater generated in association with inspections, remaining water of RO concentrated salt water, etc. An inspection for System B will be conducted after Systems A and C have been inspected.
- To reduce the risks of strontium-treated water, treatment by additional and high-performance multi-nuclide removal equipment is underway (additional from May 27, high-performance from April 15). As of June 18, approx. 26,000 m³ had been treated.
- On June 11, an incident occurred at the additional multi-nuclide removal equipment, in which successive standby operation was automatically suspended following the change in control software. There was no leakage of contaminated water or device failure due to the impact of the automatic suspension. The identified cause was grounding of the source circuit for the control equipment. After confirming the soundness of the source circuit and recovering the power source, treatment resumed on June 13. Investigations continue to determine the causes of the grounding and the impact of the software modification.
- \geq Status of investigation on accumulated water detected around the outer periphery of the HIC hatch
- In response to the detection of accumulated water around the outer periphery of the HIC* hatch on April 2, investigations to confirm any other accumulated water is being conducted. On June 15, an inspection of the temporary absorption vessel storage (secondary facility) has been completed, which confirmed accumulated water at 30 HICs out of 684 HICs stored.

* High Integrity Container (HIC): A container storing sediment products (slurry) and spent adsorbent generated from the pre-treatment facilities and absorption vessels of the multi-nuclide removal equipment, etc.

- From May 29, water removal from HICs commenced at the secondary facility. As of June 24, water removal of 32 HICs has been completed.
- As the inspection revealed HICs which included hatches with no vent hole, an additional check was conducted on hatches lacking data on the number of vent holes. This check identified eight hatches with an excessive or insufficient number of holes, of 478 HICs at the secondary facility. Hatches with no holes were replaced. Holes will be added to the prescribed number for hatches lacking holes.
- Regarding HICs with rubber mats, which were added to prevent expanded contamination in response to the leak from HICs in April 2014, it was shown that vent holes did not function due to the rubber mats. These mats were collected to ensure the vent function (June 18).
- Inspection will continue at the temporary absorption vessel storage (third facility).
- Toward reducing the risk of contaminated water stored in tanks \geq
- Treatment measures comprising the removal of strontium by cesium absorption apparatus (KURION) (from January 6) and secondary cesium absorption apparatus (SARRY) (from December 26, 2014) are underway. As of June 18, approx. 68.000 m³ had been treated.

- Measures in Tank Areas
- 2014 (as of June 23, 2015 a total of 26,250 m³).
- Collection of contaminated soil around the underground reservoir No. 1 completed
- reservoir No. 2 had been collected by August 2013.
- Leakage from transfer hose from 1,000t notch tank to Unit 3 Turbine Building
- On May 29, leakage from the pressure hose was detected during the transfer from a 1,000t notch tank to the Unit 3 approx. 7-15 m³.
- Immediately, water inside the drainage channel and drainage side ditch was collected (from May 29), likewise soil sandbags were removed (June 3).
- The pressure hoses, with which leakage was detected, were replaced with polyethylene pipes (completed on June 20).
- Inspections were conducted for all pressure hoses used on site (May 30 June 10).
- It was confirmed that hoses handling highly-concentrated water were currently used under adequate management.
- · Among hoses not handling highly-concentrated water, two disabled lines, which will not be used in future, will be removed. 139 usable lines requiring improvement will be upgraded according to the plan.
- Even usable hoses which will no longer be used in future will also be removed according to the plan.
- Inflow of groundwater into Unit 2 additional FSTR, etc.
- Buildings, accumulated water within the buildings was released using temporary pumps.
- In association with the decrease in the water level of Unit 2 additional FSTR building, the location of the groundwater detected.
- Following the release of accumulated water from the building, a visual inspection will be conducted.
- FSTR.
- Installation of the central waste treatment building bypass line
- At present, accumulated water in the Turbine Building basement is temporarily stored in the basement of the central waste treatment buildings (main process building and High Temperature Incinerator Building) and nuclides of this water are removed using various types of treatment equipment. To reduce the risks of storing highly-concentrated contaminated water in the basement of the central waste treatment buildings, there is a plan to install systems to

Collection of soil and sand around the underground reservoir No. 1, which was contaminated due to the leakage of RO concentrated salt water in April 2013, was completed (June 2). Contaminated soil around the underground

Turbine Building. The leaking water was assumed to flow inside the port via the drainage side ditch and drainage channel K. However, judging from the lack of significant variation identified in radiation density at the port entrance and in the outer sea, there was not considered to be any impact outside the port. The assumed leakage amount was

and sand inside the drainage side ditch (May 29) and sandbags were installed (May 29). As the density at the K drainage channel outlet remained within the normal range (below 200Bg/L), water collection was finished and the

At Unit 2 additional FSTR* and Unit 3 FSTR, which were evaluated as having no connection with the Reactor

* FSTR (filter sludge tank room): Building for underground waste storage

inflow was identified. In association with the decrease in the water level of Unit 3 FSTR building, an increase in the water level inside the building was identified after the release, though no new spot of groundwater inflow was

During the transfer of accumulated groundwater in Unit 3 FSTR, distortion was identified at the side plate of the temporary waste sludge storage tank (A) within the building (June 18). The results of inspection inside the tank confirmed hexagonal distortion inside the tank (June 22). No damage was identified with other tanks inside Unit 3

transfer accumulated water inside the Turbine Building basement directly to each treatment equipment. An application for this change in the implementation plan was submitted on June 9.

- \geq Removal of contaminated water from seawater-pipe trenches
- Regarding the Unit 2 seawater-pipe trench, filling of the tunnel sections was completed on December 18, 2014. Filling of Vertical Shafts A and D commenced on February 24, 2015 and the first and second filling cycles were completed on April 7 and May 27 respectively. Filling of open-cut ducts commenced on June 3. Filling of Vertical Shaft C commenced on June 1 and was completed on June 22. Removal of accumulated water is scheduled for completion within June.
- Regarding the Unit 3 seawater-pipe trench, filling of the tunnel sections had been completed (from February 5 to April 8). Water-pumping tests were conducted to verify the filling of these tunnel sections (April 16, 21 and 27), which confirmed the lack of communication in Tunnels. Filling of Vertical Shafts D and A commenced on May 2 and 15 respectively. Filling of Vertical Shaft A was completed on June 6. Filling of Vertical Shafts B and C commenced on June 13 and 17 respectively. Removal of accumulated water will commence as soon as the preparation for transfer to Unit 1 condensate storage tank is completed.
- Regarding the Unit 4 seawater-pipe trench, filling of the tunnel sections (February 14 March 21) and opening apertures II and III (April 15 - 28) had been completed. Filling of the upper stream of drainage channels will be conducted after coordinating with other construction nearby. Opening apertures I will be filled when the contaminated water level of the building is reduced.
- Regarding the overall contaminated water removal from seawater-pipe trenches, approx. 83% had been completed (as of June 23).

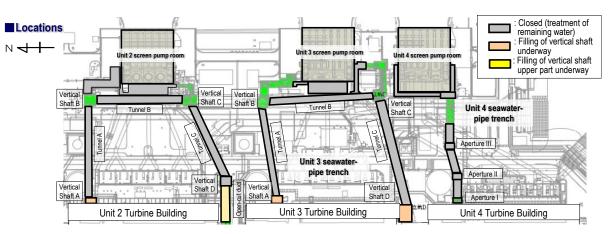


Figure 5: Progress status of contaminated water removal of seawater-pipe trenches

2. Fuel removal from the spent fuel pools

Work to help remove spent fuel from the pool is progressing steadily while ensuring seismic capacity and safety. The removal of spent fuel from the Unit 4 pool commenced on November 18, 2013 and was completed on December 22, 2014.

- Main work to help remove spent fuel at Unit 1
- On May 21, a disconnect was found with the balloon installed on the Reactor Building 3rd floor equipment hatch to reduce the release of radioactive materials. The investigative results on the status showed that rubble and anti-scattering agents had accumulated on the rain cover over the balloon and that there was no damage with the balloon. As the latest data also showed that the release amount was sufficiently low without a balloon, it will not be recovered. However, countermeasures will be taken to reduce wind inflow with the aim of controlling the release.
- The removal of the building cover is being conducted with anti-scattering measures steadily implemented and safety prioritized above all.
- > Main work to help remove spent fuel at Unit 2
- To help remove spent fuel from the pool of Unit 2 Reactor Building, a yard is being constructed around the Reactor

Building to ensure a work area for installing large heavy-duty machines.

- · Currently, preparatory works including closure of ducts, etc. and transfer of existing facilities are underway. 2015.
- Main work to help remove spent fuel at Unit 3
- On May 9, failure in a zoom function was found in two monitoring cameras of the crawler crane used to remove the crawler crane (June 1-19).
- · Removal of large rubble within the spent fuel pool has resumed since June 22. The fuel-handling machine will be removed in late July.

3. Fuel debris removal

In addition to decontamination and shield installation to improve PCV accessibility, technology was developed and data gathered as required to prepare to remove fuel debris (such as investigating and repairing PCV leak locations)

- Preparation for investigation inside the Unit 2 PCV
- As preparation to investigate the status of the platform inside the Unit 2 PCV pedestal was scheduled for August, device will be inserted, commenced on June 11 through remote operation.
- Following the removal of these blocks, construction for X-6 penetration will be conducted in July.

4. Plans to store, process and dispose of solid waste and decommission of reactor facilities

Promoting efforts to reduce and store waste generated appropriately and R&D to facilitate adequate and safe storage, processing and disposal of radioactive waste

- Management status of rubble and trimmed trees
- As of the end of May, the total storage volume of concrete and metal rubble was approx. 155,100 m³ (+3,600 m³) attributable to construction related to facing.
- Management status of secondary waste from water treatment
- As of June 18, 2015, the total storage volume of waste sludge was 597 m³ (area-occupation rate: 85%) and that of High-Integrity Containers (HICs) for multi-nuclide removal equipment, etc. was 2, 571 (area-occupation rate: 42%).
- Start of excavation for 9th solid waste storage
- For solid waste storage with a capacity of approx. 110,000 200L-drums, earth-retaining work in association with the excavation commenced on June 8.
- Acceptance of rubble started at the 3rd temporary soil cover type storage commenced
- From June 23, acceptance of rubble started at the 3rd temporary soil cover type storage, which was installed to store concrete and metal rubble with a surface dose rate of 30 mSv/h or less.

5. Reactor cooling

The cold shutdown condition will be maintained by cooling the reactor by water injection and measures to complement status monitoring will continue

Reinstallation of the water level gauge and thermometer inside Unit 1 PCV

Following completion of these works, full-scale removal of interfering buildings will commence from around August,

rubble. One of the two cameras was replaced (May 13) and the other was repaired during the annual inspection of

removal of shielding blocks installed in front of the PCV penetration (X-6 penetration), from which the investigation

compared to at the end of April, with an area-occupation rate of 62%). The total storage volume of trimmed trees was approx. 82,500 m³ (+3,900 m³ compared to at the end of April, with an area-occupation rate of 60%). The increase in rubble was mainly attributable to construction related to facing, the removal of rubble around Unit 1-4 buildings, the installation of tanks and the 9th solid waste storage. The increase in trimmed trees was mainly

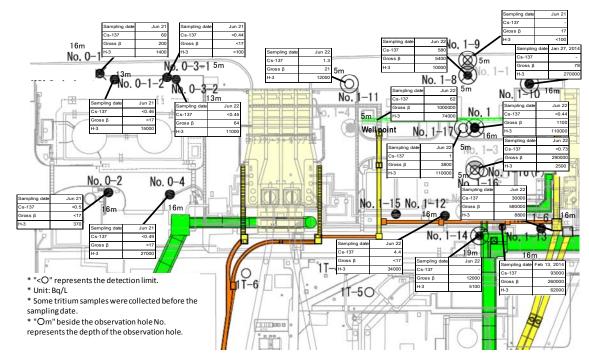
concentrated waste fluid was 9,237 m³ (area-occupation rate: 46%). The total number of stored spent vessels,

- To investigate inside the PCV, the permanent supervisory instrumentation (thermometer and water level gauge) installed there was removed (April 7). Upon completion of the investigation (April 20), the permanent supervisory instrumentation was reinstalled (April 22-23). Based on the temperature trends for one month since the installation, the instrumentation was deemed capable of being used to monitor the cooling status and surveillance of this instrumentation started on June 4.
- \geq Reading failure in the radiation detector of Unit 1 PCV gas control equipment
- On June 12 and 13, a reading failure was identified in the radiation detector of PCV gas control equipment system B after this system issued an alert. After this detector and cooling facility was replaced and inspected, the system was recovered on June 17. There was no impact on plant data monitoring because System A operated correctly while measurement of System B was suspended.
- Water purification of the Unit 1 spent fuel pool
- Radioactivity removal will be conducted from the water of the Unit 1 spent fuel pool in preparation for a case requiring desalination due to the impact of wind, rain, etc. after removing the building cover (scheduled for commencement in late July).

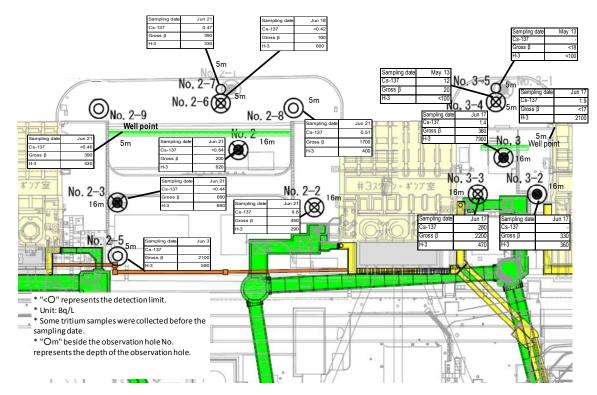
6. Reduction of radiation dose and mitigation of contamination

Effective dose-reduction at site boundaries and purification of the port water to mitigate the impact of radiation on the external environment

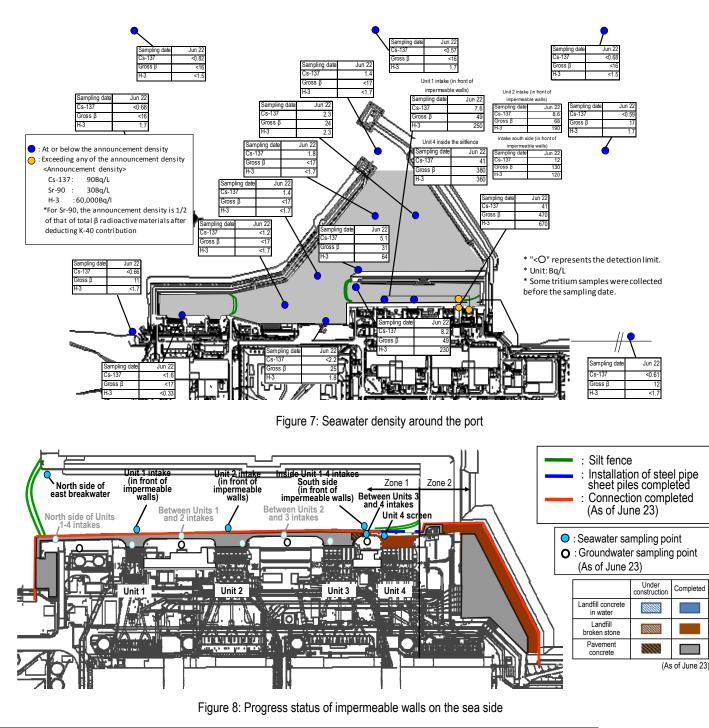
- Status of groundwater and seawater on the east side of Turbine Building Units 1 to 4 \geq
- Regarding the radioactive materials in groundwater near the bank on the north side of the Unit 1 intake, tritium densities have been increasing in groundwater Observation Hole Nos. 0-4 since July 2014 and currently stand at around 25,000 Bq/L. Pumping of 1 m³/day of water from Observation Hole No. 0-3-2 continues.
- Regarding the groundwater near the bank between the Unit 1 and 2 intakes, the tritium density at groundwater Observation Hole Nos. 1 and 1-17 has remained constant at around 120,000 Bg/L since March 2015. The density of gross β radioactive materials at groundwater Observation Hole No. 1 has been increasing since February 2015 and currently stands at around 1,000 Bq/L, while the density at groundwater Observation Hole No. 1-17 has been decreasing and currently stands at around 4,000 Bg/L. Water pumping from the well point (10m³/day) and the pumping well No. 1-16 (P) (1m³/day) installed near Observation Hole Nos. 1-16 continues.
- Regarding radioactive materials in the groundwater near the bank between the Unit 2 and 3 intakes, the densities of tritium and gross ß radioactive materials have been further decreasing from March and currently stand at around 500 Bg/L for both tritium and gross β radioactive materials. To treat the surface of the ground improvement area and repair the well point, the volume of water pumped from the well point increased to 50 m³/day (from October 31, 2014). The surface treatment commenced on January 8 and was concluded on February 18. The repair of the well point is underway (scheduled for completion in mid-July).
- Regarding the radioactive materials in groundwater near the bank between the Unit 3 and 4 intakes, a low density was maintained at all observation holes. Following the surface treatment in the ground improvement area (March 19-31), pumping of rising groundwater commenced (20m³/day from April 1, 10m³/day from April 24). Both densities of tritium and gross β radioactive materials have been increasing at groundwater Observation Hole No. 3 since April. The repair of the well point is underway (scheduled for completion in mid-July).
- Regarding the radioactive materials in seawater outside the sea side impermeable walls and within the open channels of Units 1-4, a low density equivalent to that at the point north of the east breakwater was maintained as up to May.
- The density of radioactive materials in seawater within the port has remained low at the same level as up to May.
- The radioactive material density in seawater at and outside the port entrance has remained within the same range as previously recorded.







<Between Unit 2 and 3 intakes, between Unit 3 and 4 intakes> Figure 6: Groundwater density on the Turbine Building east side



7. Review of the number of staff required and efforts to improve the labor environment and conditions

Securing appropriate staff long-term while thoroughly implementing workers' exposure dose control. Improving the work environment and labor conditions continuously based on an understanding of workers' on-site needs

Staff management

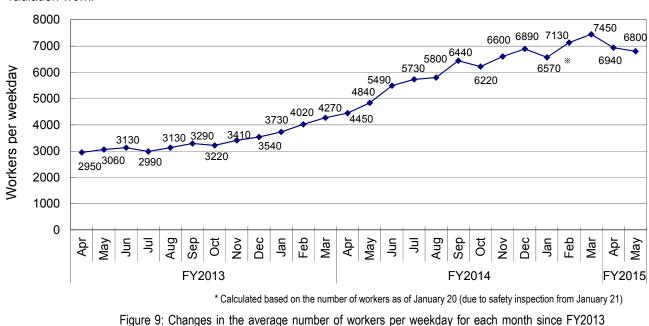
- The monthly average total of people registered for at least one day per month to work on site during the past quarter from February to April 2015 was approx. 15,000 (TEPCO and partner company workers), which exceeded the monthly average number of actual workers (approx. 11,900). Accordingly, sufficient people are registered to work on site.
- It was confirmed with prime contractors that the estimated manpower necessary for the work in July (approx. 6,800 per day: TEPCO and partner company workers)* would be secured at present. The average numbers of workers per day for each month (actual values) were maintained, with approx. 3,000 to 7,500 since FY2013 (See Figure 9).

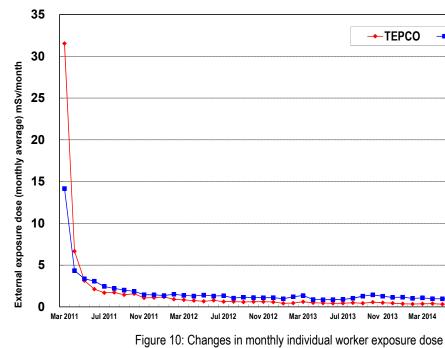
* Some works for which contractual procedures have vet to be completed are excluded from the July estimate.

• The number of workers from Fukushima Prefecture has remained the same but workers from outside the prefecture has been slightly decreasing. Therefore, the local employment ratio (TEPCO and partner company workers) as of

May remained at around 45% with a slight increase.

- The average exposure dose of workers remained at approx. 1 mSv/month during both FY2013 and FY2014. (Reference: Annual average exposure dose 20mSv/year = 1.7mSv/month)
- · For most workers, the exposure dose was sufficiently within the limit and allowed them to continue engaging in radiation work.





Status of heat stroke cases

- As of June 24 in FY2015, there were a total of five heat stroke cases, four of which attributable to work and one there were a total of five heat stroke cases, one of which attributable to work and alleged cases.)
- Renovation of dining space at the large rest house
- · At the large rest house with a capacity of approx. 1,200 workers, the operation started on May 31 and meal services at the dining room commenced on June 1.
- Regarding the meal service, having decided that renovation would be required for a portion of the building to

		→ TEP	- 00	- Partn	er comp	any	
				April 2015			
				Average 0.80 mSv (provisional value)			
				u			
							
2013	Jul 2013	Nov 2013	Mar 2014	Jul 2014	Nov 2014	Mar 2015	

(monthly average exposure dose since March 2011)

alleged case. Through preventive measures for heat stroke continue to be taken. (As of the end of June FY2014,

continue long-term operation and further improve it from a hygiene perspective, the dining room has been

temporarily closed from June 9 to 23. The renovation included enhancing the ceiling, installing additional hand-wash stations and establishing a container carry-in entrance. As the construction schedule was decided, operations temporarily resumed on June 24. However, the operations will be suspended again for about three weeks from June 29 and resume from late July. During the suspension period, the service hours of the dining room at the new Administration Office Building will be extended to enhance usability for workers.

8. Status of Units 5 and 6

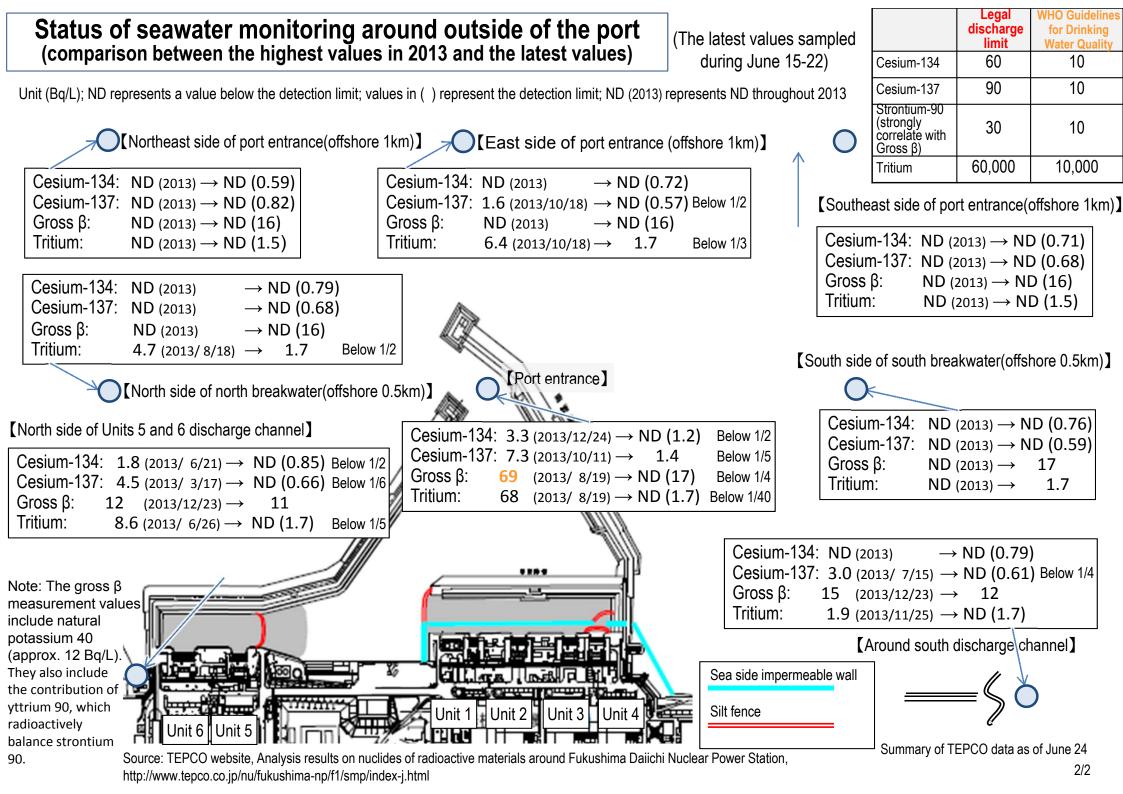
- Status of Units 5 and 6 spent fuel storage
- For Unit 5, fuel removal of the reactor commenced on April 22 and was completed on June 1. At the spent fuel pool (with a capacity of 1,590 fuel assemblies), 1,374 spent and 168 non-irradiated fuel assemblies are currently stored.
- For Unit 6, fuel removal of the reactor has been completed in FY2013. At the spent fuel pool (with a capacity of 1,654 fuel assemblies), 1,456 spent and 198 non-irradiated fuel assemblies (among which 180 fuel assemblies were transferred from Unit 4 spent fuel pool) and at the non-irradiated fuel storage (with a capacity of 230 fuel assemblies), 230 non-irradiated fuel assemblies are currently stored.
- > Status of Unit 5 and 6 accumulated water treatment
- Accumulated water in Unit 5 and 6 buildings is transferred from Unit 6 Turbine Building to outdoor tanks. After separating oil, providing RO treatment and confirming the radiation density, the water is sprinkled.

9. Other

- Revision of the Mid-and-Long-Term Roadmap
- On June 12, at the Inter-Ministerial Council for Contaminated Water and Decommissioning Issues, revision of the Mid-and-Long-Term Roadmap was resolved.
- The overall goal, including "completion of decommissioning 30-40 years from now", etc. was maintained. Based on this framework, the following five fields were revised:
- i) Emphasizing risk reduction (as well as increasing speed, prioritizing risks to ensure steady long-term risk reduction)
- ii) Specifying targets (milestones) (in response to the views of local stakeholders, setting specific targets over the next several years)
- iii) Enhancing trusting relationships with local citizens through thorough information disclosure, etc. (enhancing communications)
- iv) Further reducing the exposure dose of workers and strengthening the labor, safety and hygiene control
- v) Reinforcing the Nuclear Damage Compensation and Decommissioning Facilitation Corporation (NDF) (centrally managing research and development and accumulating knowledge domestically and overseas)
- Based on this revised Mid-and-Long-Term Roadmap, decommissioning and contaminated water countermeasures will steadily proceed.
- Meeting of the Fukushima Advisory Board on Decommissioning and Contaminated Water Management
- On June 15, at the 8th meeting of the Fukushima Advisory Board on Decommissioning and Contaminated Water Management (Fukushima City), the revised Mid-and-Long-Term Roadmap, which was approved by the Inter-Ministerial Council for Contaminated Water and Decommissioning Issues on June 12, was explained.
- During the meeting, the revised Mid-and-Long-Term Roadmap was positively evaluated for shifting its priority on ensuring safety rather than speed. Requests were also received for appropriate information delivery concerning decommissioning and contaminated water countermeasures.
- The decommissioning and contaminated water countermeasures will steadily proceed based on continued close communication with local citizens and their requests.

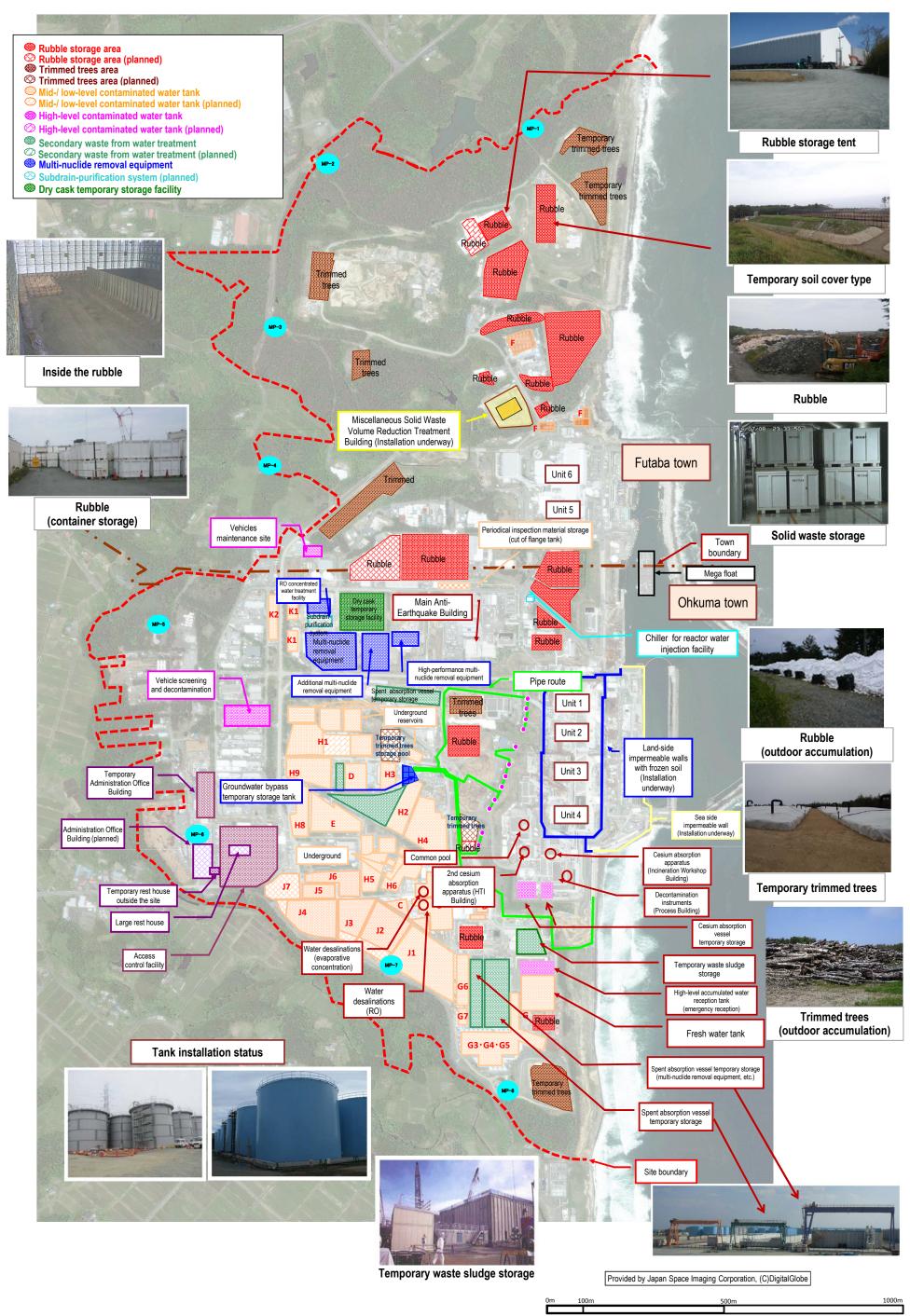
Appendix 1

Status of seawater monitoring within the port (comparison between the highest values in 2013 and the latest values) "The highest value" → "the latest value (sampled during June 15-22)"; unit (Bq/L); ND represents a value below the detection limit Sea side impermeable wall Source: TEPCO website Analysis results on nuclides of radioactive materials around Fukushima Daiichi Nuclear Cesium-134: 3.3 (2013/10/17) \rightarrow ND(1.1) Below 1/3 Power Station http://www.tepco.co.jp/nu/fukushima-np/f1/smp/index-j.html Silt fence Cesium-137: 9.0 (2013/10/17) → 1.8 Below 1/5 Cesium-134: ND(1.3) Gross β: (2013/ 8/19) → ND(17) 74 Below 1/4 Cesium-134: 3.3 $(2013/12/24) \rightarrow ND(1.2)$ Below 1/2 Cesium-137: 5.1 (2013/ 8/19) → ND(1.7) Below 1/30 Tritium: 67 Cesium-137: 7.3 (2013/10/11) → 1.4 Below 1/5 Gross β : 31 Gross B: **69** $(2013/8/19) \rightarrow ND(17)$ Below 1/4 Tritium: 64 Below 1/3 Cesium-134: 4.4 (2013/12/24) \rightarrow ND(1.3) Tritium: 68 $(2013/8/19) \rightarrow ND(1.7)$ Below 1/40 Cesium-137: 10 (2013/12/24) → 1.4 Below 1/7 Cesium-134: 3.5 (2013/10/17) → ND(1.3) Gross β: 60 $(2013/7/4) \rightarrow ND(17)$ Below 1/3 Below 1/2 Port entrance Cesium-137: 7.8 (2013/10/17) → Tritium: $(2013/8/19) \rightarrow ND(1.7)$ Below 1/30 59 2.3 Below 1/3 Gross β: 79 (2013/ 8/19) → 24 Below 1/3 Cesium-134: 5.0 $(2013/12/2) \rightarrow ND(1.4)$ Below 1/3 Tritium: 60 (2013/ 8/19) → 14 Below 1/4 Below 1/7 Cesium-137: 8.4 $(2013/12/2) \rightarrow ND(1.2)$ Cesium-134: 32 (2013/10/11) → Below 1/20 2.5 Gross β: 69 Below 1/4 $(2013/8/19) \rightarrow ND(17)$ South side Cesium-137: 73 (2013/10/11) → 8.2 Below 1/7 Tritium: Below 1/30 52 $(2013/8/19) \rightarrow ND(1.7)$ in the port Gross β: 320 (2013/ 8/12) → 49 Below 1/2 Cesium-134: 2.8 (2013/12/2) → ND(1.5) Tritium: 510 (2013/ 9/ 2) → Below 1/2 Below 6/10 230 Cesium-137: 5.8 $(2013/12/2) \rightarrow ND(1.6)$ Below 1/3 [East side in the port] Cesium-134: 3.4 3.9 Cesium-134: Gross β: 46 $(2013/8/19) \rightarrow ND(17)$ Below 1/2 Cesium-137: 7.6 Cesium-137: 8.6 Gross β : Gross B: Tritium: 49 68 $(2013/8/19) \rightarrow ND(3.3)$ 24 Below 1/7 [Port center] 250 Tritium: Tritium: 190 * WHO West side Cesium-134: ND(2.4) Legal Guidelines for discharge in the port] Cesium-137: 12 Drinking limit Gross B: 130 Water Quality Tritium: 120 60 10 Cesium-134 [North side in the port] Monitoring commenced in or after 10 In front of shallow 90 Cesium-137 March 2014 [In front of Unit 6 intake] draft quay Strontium-90 (strongly 30 10 Cesium-134: **62** (2013/ 9/16)→ 9.1 Below 1/6 correlate with Cesium-137: 140 (2013/ 9/16)→ 41 Below 1/3 Gross β) 60.000 10.000 Gross β: Tritium 360 (2013/ 8/12)→ 380 Tritium: 400 (2013/ 8/12)→ 360 Cesium-134: $5.3(2013/8/5) \rightarrow ND(1.5)$ Below 1/4 Cesium-134: 28 (2013/ 9/16)→ 12 Below 1/2 Note: The gross β measurement Cesium-137: 8.6 (2013/8/ 5) \rightarrow ND(2.2) Below 1/3 Cesium-137: 53 (2013/12/16)→ 41 values include natural potassium 40 Below 8/10 (approx. 12 Bq/L). They also include Summary of Gross β: 40 $(2013/7/3) \rightarrow 25$ Below 7/10 470 Gross B: **390** (2013/ 8/12)→ the contribution of yttrium 90, which TEPCO data as Tritium: 340 1.8 Below 1/100 (2013/6/26) → 650 (2013/ 8/12)→ 670 Tritium: radioactively balance strontium 90. of June 24



Appendix 2 June 25, 2015

TEPCO Fukushima Daiichi Nuclear Power Station Site Layout

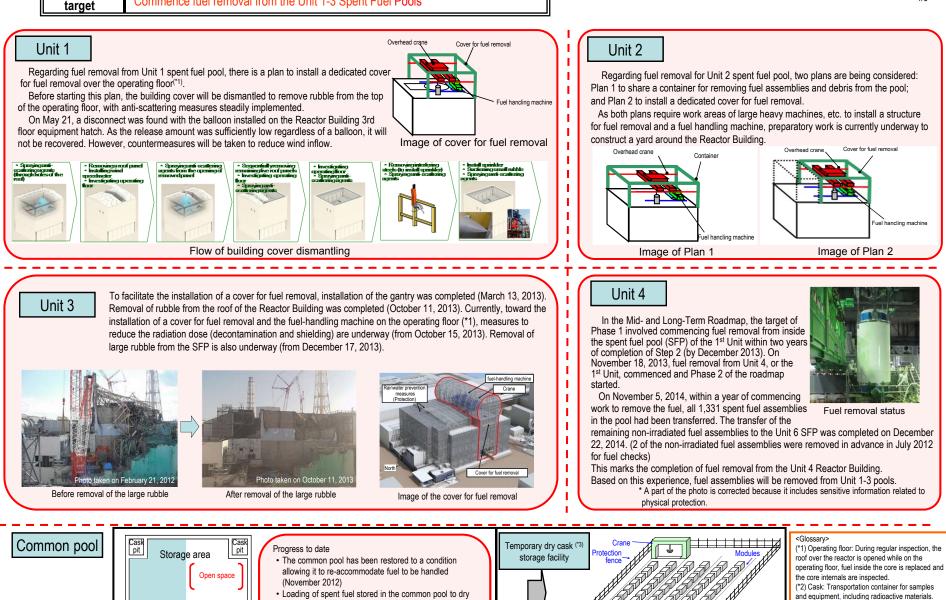


Reference June 25, 2015

Progress toward decommissioning: Fuel removal from the spent fuel pool (SFP)

Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment 1/6





Spent fuel is accepted from the common pool

Operation commenced on April 12, 2013; from the cask-storage building, transfer of 9 existing dry casks completed

(May 21, 2013); fuel stored in the common pool sequentially transferred

casks commenced (June 2013)

be received (November 2013)

An open space will be maintained in

the common pool (Transfer to the temporary dry cask storage facility)

· Fuel removed from the Unit 4 spent fuel pool began to

Identify the plant status and commence R&D and decontamination toward fuel debris removal

Immediate

target

Val June 25, 2015 Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment 2/6

3D laser scan inside the Unit 1 R/B Investigation in the leak point detected in the upper part of underground floor Unit 1 Suppression Chamber (S/C^(*1)) Investigation in the leak point detected in the upper part of Unit 1 S/C from May 27, 2014 from The upper part of the underground floor (torus room) of Unit 1 one expansion joint cover among the lines installed there. As no leakage was identified R/B was investigated with a laser scan using a remote-controlled from other parts, specific methods will be examined to halt the flow of water and repair the PCV. robot, and collected 3D data. 3D data, which allows examination based on actual External appearance of Image of investigation remote-controlled robot measurements, can be used to examine more detailed Direction of PCV accessibility and allocation of equipment. Combining it with 3D data on the R/B 1st floor allows obstacles on both 1st and underground floors to be checked simultaneously. 270° side This allows efficient examination of positions to install repair Vacuum break line E Image of the S/C upper part investigation Leak point equipment for PCVs and vacuum break lines. Image of 3D data Unit 1 Status of equipment development toward investigating inside the PCV Prior to removing fuel debris, to check the conditions inside the Primary Containment Vessel (PCV), including the Air dose rate inside the Reactor Building: location of the fuel debris, investigation inside the PCV is scheduled. Max. 5.150mSv/h (1F southeast area) (measured on July 4, 2012) [Investigative outline] Reactor Building Inserting equipment from Unit 1 X-100B penetration^(*5) to investigate in clockwise and counter-clockwise directions. [Status of investigation equipment development] Nitrogen injection flow Using the crawler-type equipment with a shape-changing structure which allows it to enter the PCV from the narrow access entrance rate into the RPV(*3): Building cover (bore: 100mm) and stably move on the grating, a field demonstration was implemented from April 10 to 20. 2015. 27.46Nm3/h Through this investigation, information including images inside the PCV 1st floor and airborne radiation was obtained. The investigation also confirmed the absence of obstacles around the access aperture leading to the basement floor, which will be used in the next investigation. These results will be considered in next investigation of the PCV basement floor. SFP (*2) temperature: 22.5°C 392 Reactor feed water system: 2.2m3/h Core spray system: 2.0m3/h Temperature inside the PCV: approx. 24°C Temperature of the RPV Investigative equipment bottom: approx. 24°C PCV hydrogen concentration System A: 0.00vol%. System B: 0.00vol% Nitrogen injection flow rate into the PCV(*4): -Nm3/h Water level of the torus room: approx. OP3.700 Air dose rate inside the PCV: 4.1 - 9.7Sv/h (measured on February 20, 2013) (Measured from April 10 to Air dose rate inside the torus room: 19, 2015) approx. 180-920mSv/h Temperature inside the PCV: approx. 25°C Water level inside the PCV (measured on February 20, 2013) <Glossarv> PCV bottom + approx. 2.8m (*1) S/C (Suppression Chamber): Temperature of accumulated water inside Suppression pool, used as the water source for Water level at the triangular corner: OP3,910-4,420 the torus room: approx. 20-23°C the emergent core cooling system. (measured on September 20, 2012) (measured on February 20, 2013) (*2) SFP (Spent Fuel Pool): Temperature at the triangular corner: 32.4-32.6°C (*3) RPV (Reactor Pressure Vessel) Water level of the Turbine Building: OP2,652 (measured on September 20, 2012) (*4) PCV (Primary Containment Vessel) Investigation inside PCV (*5) Penetration: Through-hole of the PCV **Turbine Buildina** * Indices related to the plant are values as of 11:00, June 24, 2015

Progress toward decommissioning: Works to identify the plant status and toward fuel debris removal

Immediate target

Unit 2

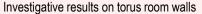
Identify the plant status and commence R&D and decontamination toward fuel debris removal

JV2I June 25, 2015 Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment 3/6

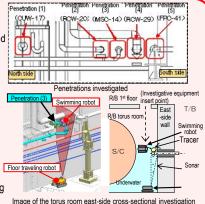
Installation of an RPV thermometer and permanent PCV supervisory instrumentation (1) Replacement of the RPV thermometer

- As the thermometer installed at the Unit 2 RPV bottom after the earthquake had broken, it was excluded from the monitoring thermometers (February 19, 2014).
- On April 17, 2014, removal of the broken thermometer failed and was suspended. Rust-stripping chemicals were injected and the broken thermometer was removed on January 19, 2015. A new thermometer was reinstalled on March 13, 2015. The thermometer has been used as a part of permanent supervisory instrumentation since April 23.
- (2) Reinstallation of the PCV thermometer and water-level gauge
- Some of the permanent supervisory instrumentation for PCV could not be installed in the planned locations due to interference with existing grating (August 13, 2013).
- The instrumentation was removed on May 27, 2014 and new instruments were reinstalled on June 5 and 6, 2014. The trend of added instrumentation will be monitored for approx. one month to evaluate its validity.
- The measurement during the installation confirmed that the water level inside the PCV was approx. 300mm from the bottom.

Removal situation of broken thermometer inside Unit 2 RPV



- The torus room walls were investigated (on the north side of the east-side walls) using equipment specially developed for that purpose (a swimming robot and a floor traveling robot).
- At the east-side wall pipe penetrations (five points), "the status" and "existence of flow" were checked.
- A demonstration using the above two types of underwater wall investigative equipment showed how the equipment could check the status of penetration.
- Regarding Penetrations 1 5, the results of checking the sprayed tracer (¹⁵) by camera showed no flow around the penetrations. (investigation by the swimming robot)
- Regarding Penetration 3, a sonar check showed no flow around the penetrations. (investigation by the floor traveling robot)



Status of equipment development toward investigating inside the PCV

Prior to removing fuel debris, to check the conditions inside the Primary Containment Vessel (PCV), including the location of the fuel debris, investigations inside the PCV are scheduled.

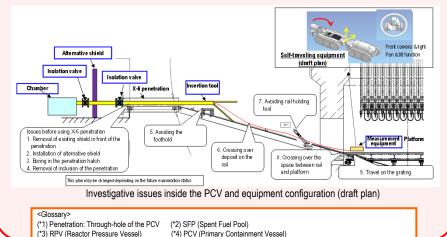
[Investigative outline]

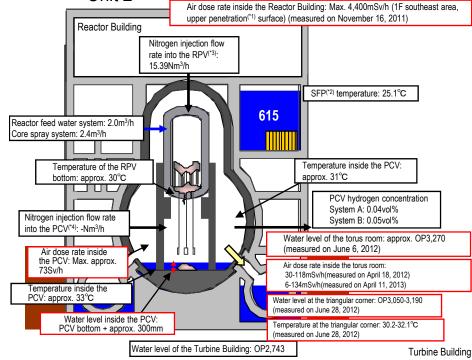
 Inserting the equipment from Unit 2 X-6 penetration^(*1) and accessing inside the pedestal using the CRD rail to conduct investigation.

[Status of investigative equipment development]

(*5) Tracer: Material used to trace the fluid flow. Clav particles

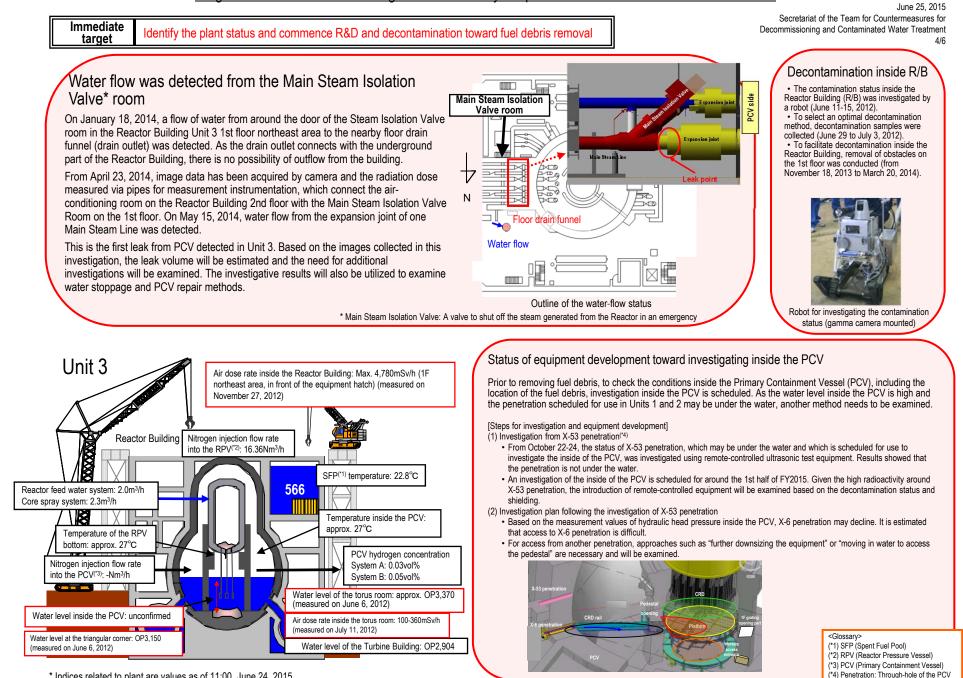
 Based on issues confirmed by the CRD rail status investigation conducted in August 2013, the investigation method and equipment design are currently being examined. A demonstration is scheduled in the field in the 1st half of FY2015.





* Indices related to plant are values as of 11:00, June 24, 2015



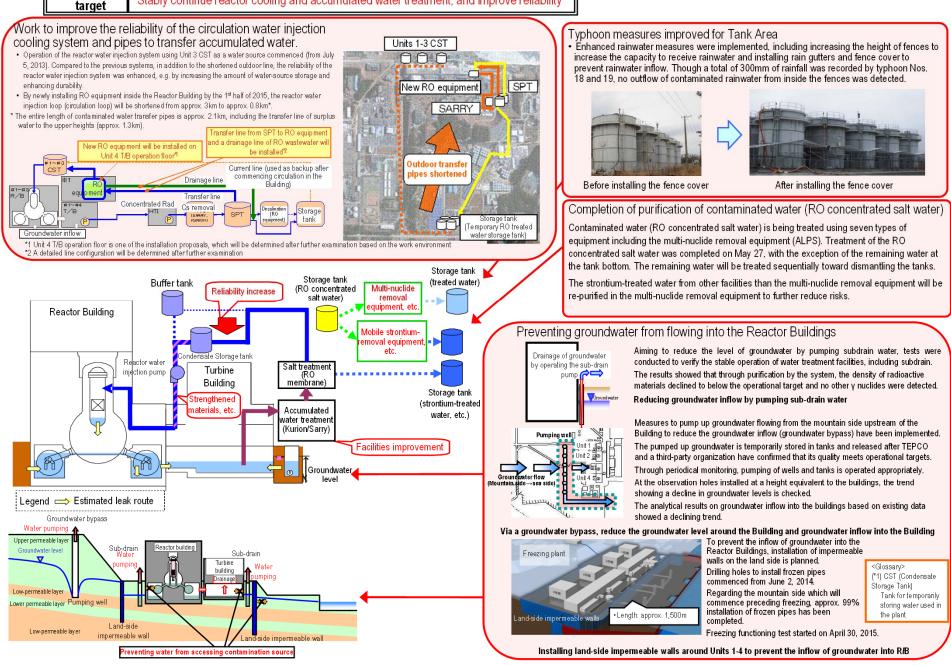


* Indices related to plant are values as of 11:00. June 24, 2015

Progress toward decommissioning: Work related to circulation cooling and accumulated water treatment line

June 25, 2015 Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment 5/6

Immediate Stably continue reactor cooling and accumulated water treatment, and improve reliability



June 25, 2015 Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment

6/6

Progress toward decommissioning: Work to improve the environment within the site

Immediate targets - Reduce the effect of additional release from the entire power station and radiation from radioactive waste (secondary water treatment waste, rubble, etc.) generated after the accident, to limit the effective radiation dose to below 1mSv/year at the site boundaries. - Prevent contamination expansion in sea, decontamination within the site

