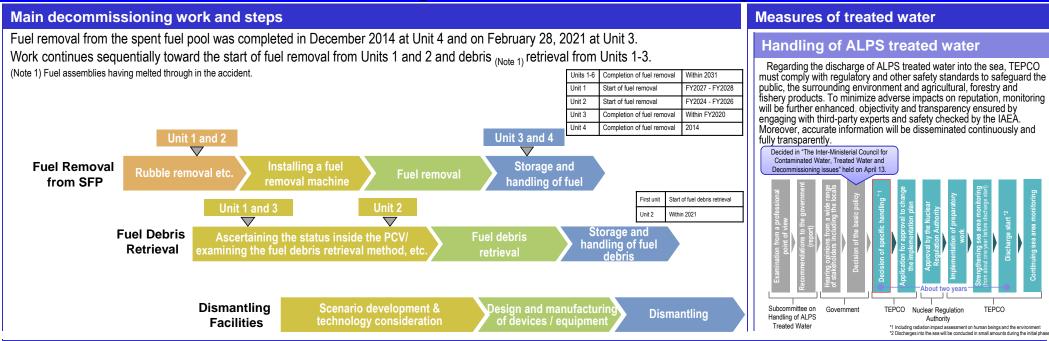
Decommissioning and Contaminated Water Treatment



Contaminated water management - triple-pronged efforts -

- (1) Efforts to promote contaminated water management based on the three basic policies

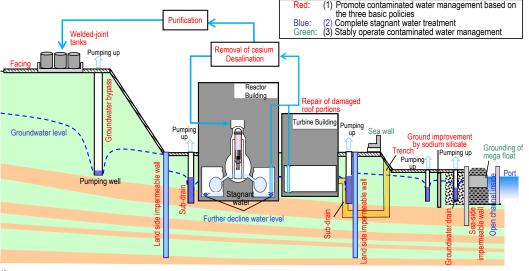
 ① "Remove" the source of water contamination ② "Redirect" fresh water from contaminated areas
- ③ "Retain" contaminated water from leakage
- Strontium-reduced water from other equipment is being re-treated in the multi-nuclide removal equipment (ALPS) and stored in welded-joint tanks.
- Multi-layered contaminated water management measures, including land-side impermeable walls and sub-drains, have stabilized the groundwater at a low level and the increased contaminated water generated during rainfall is being suppressed by repairing damaged portions of building roofs, facing onsite, etc. Through these measures, the generation of contaminated water was reduced from approx. 540 m³/day (in May 2014) to approx. 180 m³/day (in FY2019) and approx. 140 m³/day (in 2020).
- Measures continue to further suppress the generation of contaminated water to 100 m3/day or less within 2025.

(2) Efforts to complete stagnant water treatment

- To lower the stagnant water levels in buildings as planned, work to install additional stagnant
 water transfer equipment is underway. At present, the floor surface exposure condition can be
 maintained except for the Unit 1-3 Reactor Buildings, Process Main Building and the High
 Temperature Incinerator Building.
- In 2020, treatment of stagnant water in buildings was completed, except for the Unit 1-3 Reactor Buildings, Process Main Building and High-Temperature Incinerator Building. For Reactor Buildings, the amount of stagnant water there will be reduced to about half the amount at the end of 2020 during the period FY2022-2024.
- For Zeolite sandbags on the basement floors of the Process Main Building and High-Temperature Incinerator Building, measures to reduce the radiation dose are being examined with stabilization in mind.

(3) Efforts to stably operate contaminated water management

To prepare for tsunamis, various measures are underway. For heavy rain, sandbags are being
installed to suppress direct inflow into buildings while work closing building openings and
installing sea walls to enhance drainage channels and other measures are being implemented
as planned.



Progress status

- ◆ The temperatures of the Reactor Pressure Vessel (RPV) and Primary Containment Vessel (PCV) of Units 1-3 have been maintained within the range of approx. 20-35°C*1 over the past month. There was no significant change in the concentration of radioactive materials newly released from Reactor Buildings into the air*2. It was concluded that the comprehensive cold shutdown condition had been maintained.
- * 1 The values varied somewhat, depending on the unit and location of the thermometer.
- * 2 In June 2021, the radiation exposure dose due to the release of radioactive materials from the Unit 1-4 Reactor Buildings was evaluated at less than 0.00003 mSv/year at the site boundary. The annual radiation dose from natural radiation is approx. 2.1 mSv/year (average in Japan)

Mockup for decontamination completed toward Unit 2 fuel removal

Regarding the top floor of the Reactor Building, the dose evaluation showed a reduction of about 20% compared to the results in FY2018 due to containment of remaining objects which had been implemented previously.

To further reduce the dose, mockup of decontamination was conducted in the Remote Technology Development Center (in Naraha town). Subsequently, decontamination on the top floor will

Based on the effect of dose reduction by decontamination and shielding, efforts to reduce the dose will continue to approach the target level of 1 mSv/h.

Regarding the work prior to installing the gantry for fuel removal, trial ground improvement will be <Decontamination of mockup high-place wall>

implemented in early August.

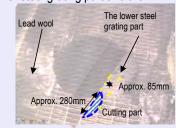
Prior to the Unit 1 PCV internal investigation. construction of the access route steadily progressing

An access route will be created before investigating inside the Unit 1 Primary Containment Vessel (PCV). Before the construction, work to cut obstacles is being implemented in three steps. On July 15, Step 2 comprising cutting the lower steel grating part and the

handrail (side part) was completed.

Step 3 comprising cutting of conduits will be implemented in mid-September.

The work proceeds carefully, with safety first and the surrounding environment unaffected.



<Cutting of lower steel grating part>

Cover for fuel removal Removed fuel (assemblies) 1535/1535*1 (Fuel removal completed on December 22, 2014)

Blowout panel (closed) Removed fuel (assemblies) Operating floor Dome roof Front chamber **566/566** Spent Fuel Pool Fuel-handling (Fuel removal completed on February 28, 2021) machine Crane FHM girder Primary ontainment Vessel (PCV) Water Water Reactor Pressure Vessel (RPV) Fuel debris Unit 4 *1 Including two new fuel assemblie removed first in 2012. Unit 2 Unit 3 Reactor Building (R/B) Unit 1

To formulate a plan of investigations inside the buildings as part of efforts to clarify the accident progress, prior investigations of Unit 1 and 2 Reactor Buildings planned

To "assume the conditions of the Unit 1-3 cores and PCVs in the Fukushima Daijchi NPS and examine the unresolved issues," efforts to clarify the accident progress continue.

In FY 2021, using a y-imager and 3D image acquisition device, information useful to formulate a plan of investigations inside the Reactor Buildings will be collected, including information on space and dose inside

the Unit 1 and 2 Reactor Buildings.

<Measurement equipment (y-imager *)> * By combining the γ -ray measurement results and 3D scan information, a 3D distribution of γ -rays is acquired

Investigation into the drilled part of the shield plug planned on the top floor of the Unit 2 Reactor Building

In April 2021, in collaboration with the Secretariat of the Nuclear Regulation Authority, the surface contamination density was assessed on the top floor of the Unit 2 Reactor Building and a significant influence from the part under the shield plug and gaps was detected.

Aiming at increasing the accuracy of the evaluation, a dose investigation using the existing drilled parts will be implemented in late August.

Results of the investigation will be utilized as input information for the accident analysis and decommissioning.



<Location of existing drilled parts>

Equipment for Unit 2 trial fuel debris retrieval arrived in Japan

Equipment for the Unit 2 trial fuel debris retrieval. which had been developed in the UK, arrived in Japan on July 10 and a domestic factory (in Kobe) on July 12.

Subsequently, performance verification tests in Japan will be conducted.

At the same time, operation training started. To learn the skills required to remotely operate the robot, nine staff members of the Fukushima Daiichi NPS were dispatched to Mitsubishi Heavy Industries, Ltd. from July 1.



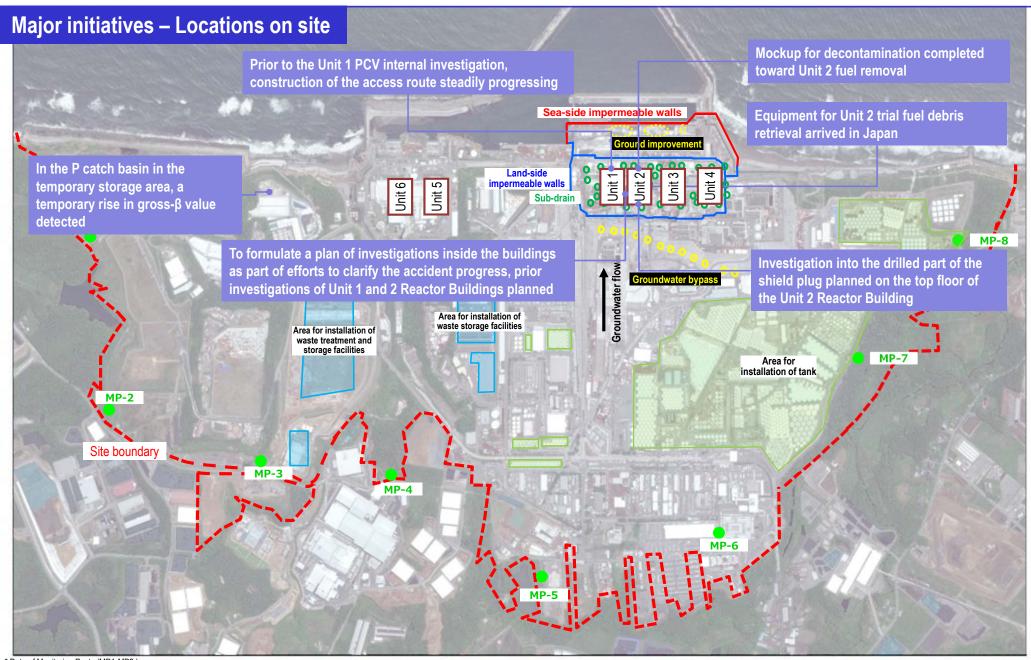
<Arrival in Japan (July 10, 2021)>

In the P catch basin in the temporary storage area, a temporary rise in gross-β value detected

On July 5, the gross-β radioactivity value in P catch basin in the temporary storage area rose temporarily and contamination was detected on the ground surface around the notch tanks there.

This was considered attributable to the top board hatch lids and the top boards of two notch tanks becoming misaligned and rainfall meaning rainwater carrying radioactive materials flowed outside. The monitoring results did not reveal any environmental influence. For those notch tanks. the top board hatch lids were recovered and sheet covers were installed to prevent rainwater inflow.

Management will also be enhanced by installing Zeolite sandbags and removing contaminated soil. Moreover, to inspect the top of notch tanks, regular drone patrols will be introduced.



^{*} Data of Monitoring Posts (MP1-MP8.)

Data (10-minute values) of Monitoring Posts (MPs) measuring the airborne radiation rate around site boundaries showed 0.316 – 1.168 µSv/h (June 23 – July 27, 2021).

We improved the measurement conditions of monitoring posts 2 to 8 to measure the air-dose rate precisely. Construction work, such as tree-clearing, surface soil removal and shield wall setting, were implemented from February 10 to April 18, 2012.

Therefore, monitoring results at these points are lower than elsewhere in the power plant site.

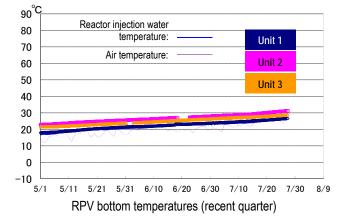
The radiation shielding panels around monitoring post No. 6, which is one of the instruments used to measure the radiation dose at the power station site boundary, were taken off from July 10 - 11, 2013, since further deforestation, etc. had caused the surrounding radiation dose to decline significantly.

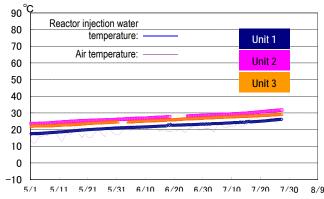
Provided by Japan Space Imaging Corp., photo taken on May 24, 2020 Product (C) [2020] DigitalGlobe, Inc., a Maxar company

I. Confirmation of the reactor conditions

Temperatures inside the reactors

Through continuous reactor cooling by water injection, the temperatures of the Reactor Pressure Vessel (RPV) bottom and the Primary Containment Vessel (PCV) gas phase were maintained within the range of approx. 20 to 35°C for the past month, though it varied depending on the unit and location of the thermometer.



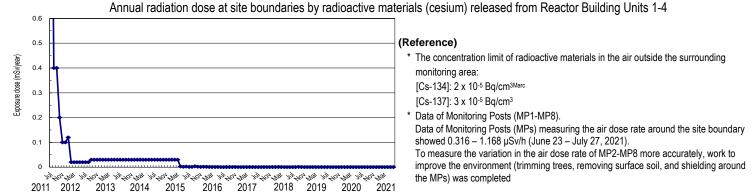


PCV gas phase temperatures (recent quarter)

- *1 The trend graphs show part of the temperature data measured at multiple points
- *2 A part of data could not be measured due to maintenance and inspection of the facility and other work.

Release of radioactive materials from the Reactor Buildings

As of June 2021, the concentration of radioactive materials newly released from Reactor Building Units 1-4 into the air and measured at the site boundary was evaluated at approx. 1.5×10^{-12} Bq/cm³ and 1.4×10^{-12} Bq/cm³ for Cs-134 and -137 respectively, while the radiation exposure dose due to the release of radioactive materials there was less than 0.00003 mSv/year.



Note 1: Different formulas and coefficients were used to evaluate the radiation dose in the facility operation plan and monthly report. The evaluation methods were integrated in September 2012. As the fuel removal from the spent fuel pool (SFP) commenced for Unit 4, the radiation exposure dose from Unit 4 was added to the items subject to evaluation since November 2013. The evaluation has been changed to a method considering the values of continuous dust monitors since FY2015, with data to be evaluated monthly and announced the following month.

Note 2: Radiation dose was calculated using the evaluation values of release amount from Units 1-4 and Units 5 and 6. The radiation dose of Unit 5 and 6 was evaluated based on expected release amount during operation until September 2019 but the evaluation method was reviewed and changed to calculate based on the actual measurement results of Units 5 and 6 from October.

Other indices

There was no significant change in indices, including the pressure in the PCV and the PCV radioactivity density (Xe-135) for monitoring criticality, nor was any anomaly in the cold shutdown condition or criticality sign detected.

Based on the above, it was confirmed that the comprehensive cold shutdown condition had been maintained and the reactors remained in a stabilized condition.

II. Progress status by each plan

Handling of ALPS treated water

- > Status of examination concerning breeding test of marine organisms
 - Regarding the breeding tests of marine organisms in treated water by multi-nuclide removal equipment and others
 (ALPS treated water), breeding tests will be conducted before starting the handling of ALPS -treated water and the
 growing conditions will be compared and evaluated under both environments of "sea water" and "ALPS treated water

- diluted by sea water."
- The tests will be conducted while obtaining the opinions and cooperation of stakeholders. Progress and other aspects will be disclosed accordingly.

Contaminated water management

Based on the three basic policies: "remove" the source of water contamination, "redirect" fresh water from contaminated areas and "retain" contaminated water from leakage, multi-layered contaminated water management measures have been implemented to stably control groundwater

Status of contaminated water generated

- Multi-layered measures, including pumping up by sub-drains and land-side impermeable walls, which were implemented to control the continued generation of contaminated water, suppressed the groundwater inflow into buildings.
- After implementing "redirecting" measures (groundwater bypass, sub-drains, land-side impermeable walls and others)
 and rainwater prevention measures, including repairing damaged portions of building roofs, the amount of
 contaminated water generated within FY2020 declined to approx. 140 m³/day.
- Measures will continue to further reduce the amount of contaminated water generated.

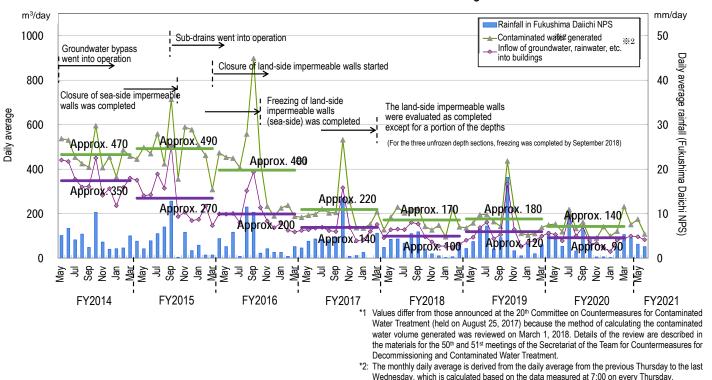


Figure 1: Changes in contaminated water generated and inflow of groundwater and rainwater into buildings

Operation of the groundwater bypass

- From April 9, 2014, the operation of 12 groundwater bypass pumping wells commenced sequentially to pump up groundwater. The release then started from May 21, 2014, in the presence of officials from the Intergovernmental Liaison Office for the Decommissioning and Contaminated Water Issue of the Cabinet Office. Up until July 28, 2021, 653,000 m³ of groundwater had been released. The pumped-up groundwater was temporarily stored in tanks and released after TEPCO and a third-party organization had confirmed that its quality met operational targets.
- Pumps are inspected and cleaned as required based on their operational status.

> Operation of the Water-Treatment Facility special for Sub-drain & Groundwater drains

To reduce the level of groundwater flowing into the buildings, work began to pump up groundwater from wells (sub-drains) around the buildings on September 3, 2015. The pumped-up groundwater was then purified at dedicated facilities and released from September 14, 2015, in the presence of officials from the Intergovernmental Liaison Office for the Decommissioning and Contaminated Water Issue of the Cabinet Office. Up until July 27, 2021, a total of

- 1,115,000 m³ had been drained after TEPCO and a third-party organization had confirmed that its quality met operational targets.
- Due to the level of the groundwater drain pond rising after the sea-side impermeable walls had been closed, pumping started on November 5, 2015. Up until July 27, 2021, a total of approx. 267,000 m³ had been pumped up and a volume of under 10 m³/day is being transferred from the groundwater drain to the Turbine Buildings (average for the period June 17 – July 21, 2021).
- As one of the multi-layered contaminated-water management measures, in addition to a waterproof pavement that aims to prevent rainwater infiltrating, facilities to enhance the sub-drain treatment system were installed and went into operation from April 2018, increasing the treatment capacity from 900 to 1,500 m³/day and improving reliability. Operational efficiency was also improved to treat up to 2,000 m³/day for almost a week during the peak period.
- To maintain the groundwater level, work to install additional sub-drain pits and recover existing pits is underway. The additional pits are scheduled to start operation sequentially, from pits for which work is completed (12 of 14 new subdrain pits went into operation). To recover existing pits, work for all three pits scheduled was completed, all of which went into operation from December 26, 2018. Work to recover another pit (No. 49) started from November 2019 and it went into operation from October 9, 2020.
- To eliminate the need to suspend water pumping while cleaning the sub-drain transfer pipe, the pipe will be duplicated. Installation of the pipe and ancillary facilities was completed.
- Since the sub-drains went into operation, the inflow to buildings tended to decline to under 150 m³/day when the subdrain water level declined below T.P. 3.0 m but increased during rainfall.

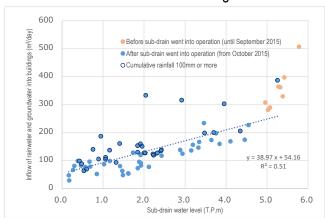


Figure 2: Correlation between inflow such as groundwater and rainwater into buildings and the water level of Units 1-4 sub-drains

Implementation status of facing

- Facing is a measure involving asphalting of the on-site surface to reduce the radiation dose, prevent rainwater infiltrating the ground and decrease the amount of underground water flowing into buildings. As of the end of June 2021, 95% of the planned area (1,450,000 m² on site) had been completed. For the area inside the land-side impermeable walls, implementation proceeds appropriately after constructing a yard from implementable zones that leave the decommissioning work unaffected. As of the end of June 2021, 25% of the planned area (60,000 m²) had been completed.
- Construction status of the land-side impermeable walls and status of groundwater levels around the buildinas
- An operation to maintain the land-side impermeable walls and prevent the frozen soil from thickening further continued from May 2017 on the north and south sides and started from November 2017 on the east side, where sufficiently thick frozen soil was identified. The scope of the maintenance operation was expanded in March 2018.
- In March 2018, construction of the land-side impermeable walls was completed, except for a portion of the depth. based on a monitoring result showing that the underground temperature had declined below 0°C in almost all areas, while on the mountain side, the difference in internal and external water levels increased to approx. 4-5 m. The 21st Committee on Countermeasures for Contaminated-Water Treatment, held on March 7, 2018, evaluated that alongside

the function of sub-drains and other measures, a water-level management system to stably control groundwater and redirect groundwater from the buildings had been established and allowed the amount of contaminated water generated to be reduced significantly.

- · A supplementary method was implemented for the unfrozen depth and it was confirmed that the temperature of this portion had declined below 0°C by September 2018. From February 2019, a maintenance operation started throughout all sections.
- The groundwater level in the area inside the land-side impermeable walls has been declining every year. On the mountain side, the difference between the inside and outside was maintained, despite varying during rainfall. The water level of the groundwater drain observation well has been maintained at approx. T.P. +1.5 m, sufficiently below the ground surface (T.P. 2.5 m).

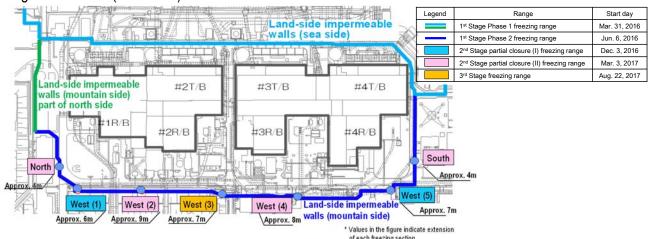
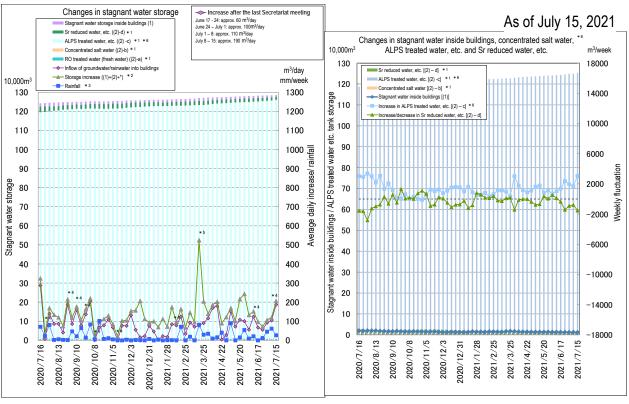


Figure 3: Closure parts of the land-side impermeable walls (on the mountain side)



^{1:} Water amount for which the water-level gauge indicates 0% or more

*3: Changed from December 13, 2018 from rainfall in Namie to that within the site.

Figure 4: Status of stagnant water storage

^{*2:} To detect storage increases more accurately, the calculation method was reviewed as follows from February 9, 2017: (The revised method was applied from March 1, 2018) [(Inflow of groundwater/rainwater into buildings) + (other transfer) + (chemical injection into ALPS)]

Considered attributable to the fluctuation inflow of groundwater, rainwater, and others to buildings due to the decline in the level of stagnant water in buildings (July 16-23, August 20-27, September 3-10 and 17-24, October 1-8, November 12-19, 2020, February 4-11, June 3-10 and July 8-15, 2021)

Stored amount increased due to transfer to buildings in association with decommissioning work on March 18, 2021. (Major breakdown of the transferred amount: (1) Stagnant water inside the tank fences (water transferred from the Shallow Draft Quay drainage channel) was transferred to the Process Main Building: approx. 390 m³/day. (2) Stagnant water inside the tank fences (water transferred from the Shallow Draft Quay drainage channel) was transferred to the High Temperature Incinerator Building: approx. 10 m³/day, (3) Transfer from the Unit 3 additional FSTR to the Unit 3 Radioactive Waste Treatment Building: approx 10 m3/day and others)

^{*6.} The notation of treated water by the multi-nuclide removal equipment and others was reviewed in accordance with the definition change of ALPS treated water by the Government (April 27, 2021)

Operation of multi-nuclide removal equipment

- Regarding the multi-nuclide removal equipment (existing and high-performance), hot tests using radioactive water are underway (for existing equipment, System A: from March 30, 2013, System B: from June 13, 2013, System C: from September 27, 2013; and for high-performance equipment, from October 18, 2014). The additional multi-nuclide removal equipment went into full-scale operation from October 16, 2017.
- As of July 15, 2021, the volumes treated by existing, additional and high-performance multi-nuclide removal equipment were approx. 476,000, 708,000 and 103,000 m³, respectively (including approx. 9,500 m³ stored in the J1(D) tank, which contained water with highly concentrated radioactive materials at the System B outlet of the existing multi-nuclide removal equipment).
- To reduce the risks of strontium-reduced water, treatment using existing, additional and high-performance multinuclide removal equipment has been underway (existing: from December 4, 2015; additional: from May 27, 2015; high-performance: from April 15, 2015). Up until July 15, 2021, approx. 807,000 m³ had been treated.

> Toward reducing the risk of contaminated water stored in tanks

 Treatment measures comprising the removal of strontium by cesium-adsorption apparatus (KURION) (from January 6, 2015), the secondary cesium-adsorption apparatus (SARRY) (from December 26, 2014) and the third cesiumadsorption apparatus (SARRY II) (from July 12, 2019) are underway. Up until July 15, 2021, approx. 645,000 m³ had been treated.

Measures in the Tank Area

• Rainwater accumulates and is collected inside the area of contaminated-water tanks. After removing radionuclides, the rainwater is sprinkled over the site grounds, if the radioactivity level does not meet the standard for discharging into the environment since May 21, 2014 (as of July 26, 2021, a total of 183,000 m³).

> Policy for measures toward eliminating the use of flanged-tanks in Unit 5 and 6 F tank area

- Leakage was detected from seven flanged-tanks in the F tank area (areas H, I and J). The leakage was considered attributable to degradation over time and malfunction of flange water-sealing materials due to vibration of earthquakes (on February 13, 2021 and others).
- As an emergency measure, the water level in those tanks was reduced to the leakage part or lower and repair by sealing materials is underway (scheduled to be implemented in August 2021).
- As preventive maintenance, the flange bellows part was sealed with putty and painted and bolt and nut parts were caulked and painted (scheduled to be implemented in FY2021).
- As mid- and long-term measures, the usage of flanged tanks will be terminated and details including risks, water balance and processes are being examined.

> Status of work to dismantle D1 and D2 (flanged-type) tanks in E area

- Regarding the remaining water inside D1 and D2 tanks (sampled on January 28, 2021), the radioactivity concentration
 was measured and the results confirmed a high concentration of α-nuclide (at a level equivalent to that of the stagnant
 water inside the buildings).
- α-nuclides are mainly particles. When the tanks which had stored RO concentrated salt water were dismantled, RO concentrated salt water (remaining water) at the bottom was collected in D1 and D2 tanks. It was considered that the sludge had settled and increased the concentration of α-nuclides in the remaining water in the lower parts of those tanks.
- On July 13 and 14, High Efficiency Particulate Air (HEPA) filters were installed on the outlets of ventilation pipes, which were located on the top of each of those tanks.
- Workers in that area wore appropriate protective equipment and no body contamination or intake were identified, nor any influence on the surrounding environment.
- The total-α concentration in the ALPS outlet water remained below the detection limit (N.D.) after ALPS treatment of that water started, showing that ALPS could remove α-nuclides to below the detection limit.

Response to the Unit 1/2 exhaust stack drain sump pit

- During the period July 14-16, 2021, as a measure to prevent rainwater flowing into the Unit 1/2 exhaust stack drain sump pit, a lid was installed over the manhole on the southeast side of the pit.
- Moreover, soil inside the manhole was sampled, within which analytical results detected high radioactivity.
- Water was sprinkled around the pit on May 17 to investigate inflow parts. Based on the results, it was considered that
 rainwater had flowed into the manhole, infiltrated the soil inside and subsequently contaminated and flowed into the
 sump pit.
- Subsequently, the water level inside the pit increased during the period of rainfall July 26-28. The investigative method of inflow route into the pit after the lid was installed over the manhole will be examined.
- Sampling and analysis of water inside the pit will continue to check for any variation in concentration before and after installing the lid.
- Regarding the Unit 1/2 exhaust stack drain sump pit, no decline in the water level was detected except for the startup time of the drainage pump, nor any leakage outside the system.

Fuel removal from the spent fuel pools

Work to help remove spent fuel from the pool is progressing steadily while ensuring seismic capacity and safety.

Main work to help spent fuel removal at Unit 1

- After examining two methods: (i) installing a cover after rubble removal and (ii) initially installing a large cover over the Reactor Building and then removing rubble inside the cover, method (ii) was selected for safer and more secure removal.
- Before removing the fallen roof and other objects on the south side, to minimize the risk of the overhead crane/fuel-handling machine shifting its position, becoming imbalanced and subsequently falling, plans were made to install materials to support the fuel-handling machine from below.
- Among the measures to prevent and alleviate rubble falling, work to install supports for the Unit 1 fuel-handling machine started from October 6, 2020 and was completed by October 23.
- To install the support for the overhead crane, preparation started from October 2020 and the work was completed on November 24.
- To install a large cover over the Reactor Building, dismantling of the building cover hindrance (remaining part) started from December 19, 2020 and was completed on June19, 2021 as originally planned.
- From late April, work to assemble a temporary gantry and others is underway in a yard outside the site prior to installing a large cover.
- A work yard is being prepared around the Reactor Building. After construction, work to install a large cover will start from the first half of FY2021.
- Rubble removal and other work will proceed steadily with safety first, toward starting fuel removal during the period FY2027 to FY2028.

Main work to help spent fuel removal at Unit 2

- After completing the training to practice work skills for transportation, preparatory work inside the operating floor started from July 20, 2020. Containers housing the remaining objects during the previous work were transported to the solid waste storage facility from August 26, 2020 which was completed by December 11, 2020.
- To reduce the dose on the operating floor, a mockup of decontamination work is underway. On-site preparation for decontamination on the operating floor will start from the end of June 2021.
- For fuel removal methods, based on investigative results inside the operating floor from November 2018 to February 2019, a method to access from a small opening installed on the south side of the building was selected with aspects such as dust management and lower work exposure in mind (the method previously examined had involved fully dismantling the upper part of the building).
- As a part of preparatory work, removal of hindrances (underground objects and others) is currently underway. Preparation and ground improvement will follow and work to install the gantry will start from the first half of FY2022.

Plans to store, process and dispose of solid waste and decommission of reactor facilities

Promoting efforts to reduce and store waste generated appropriately and R&D to facilitate adequate and safe storage, processing and disposal of radioactive waste

Management status of the rubble and trimmed trees

As of the end of June 2021, the total storage volume for concrete and metal rubble was approx. 310,700 m³ (-200 m³ compared to at the end of May with an area-occupation rate of 77%). The total storage volume of trimmed trees was approx. 136,800 m³ (+2,100m³, with an area-occupation rate of 78%). The total storage volume of used protective clothing was approx. 33,700 m³ (+700 m³, with an area-occupation rate of 49%). The decrease in rubble was mainly attributable to transfer for area arrangement, while the increase in used protective clothing was attributable to the suspension of incinerator operation.

Management status of secondary waste from water treatment

• As of July 1, 2021, the total storage volume of waste sludge was 454 m³ (area-occupation rate: 65%), while that of concentrated waste fluid was 9,380 m³ (area-occupation rate: 91%). The total number of stored spent vessels, High-Integrity Containers (HICs) for multi-nuclide removal equipment and other vessels, was 5,158 (area-occupation rate: 81%).

Revision of the Solid Waste Storage Management Plan (FY2021 version)

- The fifth revision of the "Solid Waste Storage Management Plan," which was formulated in March 2016, was issued on July 29, 2021.
- The predicted results and generation amount of "rubble and others" were calculated for around a decade from now by reflecting the results at the end of March 2021 and based on the recent construction plan and others and estimated the influence on the facility installation plan.
- Review on the construction time of the additional Radioactive Waste Incinerator and the Large Waste Storage Building
 1 was reflected and "the melting facility" was added as a decontamination and melting facility.
- The amount is anticipated to exceed the storage capacity in around FY2032. Construction of additional solid waste storage facilities will be examined. As of the end of FY2028, the storage capacity will be satisfied. Efforts will continue according to the plan toward achieving the target milestone in the Mid- and-Long-Term Roadmap "eliminating the temporary outdoor storage area within FY2028."

Status of the inspection on containers in the temporary storage area

- Regarding the appearance inspection of rubble containers with a high surface dose (0.1-30 mSv/h), 5,270 of 5,338 units had been completed as of July 27.
- Investigation of containers whose contents are not identified (including a check for moisture, 4,011 units) will start after the above appearance inspection is completed.
- Based on the results of these appearance inspections and the container investigation, the items and frequency of container inspections will be decided to implement periodical inspections.
- Hereinafter, to confirm that no moisture is included in the contents, a requirement to attach photos of the contents is added to the application for the temporary storage of containers

Reactor cooling

The cold shutdown condition will be maintained by cooling the reactor by water injection and measures to complement the status monitoring continue

Status of water-level stabilization in the Unit 1 PCV

- Since the earthquake off the coast of Fukushima Prefecture on February 13, the PCV water level had been gradually declining. Enhanced monitoring continued and the water injection rate into the reactor was changed to 3.5 m³/h and finely adjusted as necessary according to the tendency of the water level.
- As the water level subsequently almost stabilized, the water injection rate of 3.5m³/h was maintained and enhanced

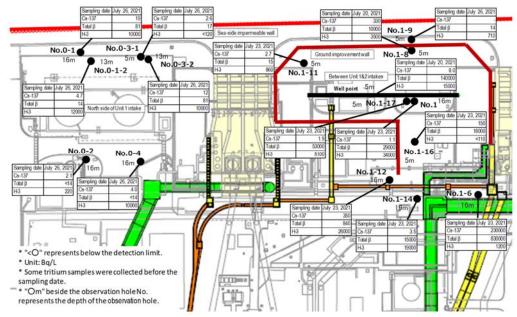
monitoring was terminated on July 16.

• It is being examined that a reduction of the water injection rate and the PCV water level of Unit 1 will be planned after FY2022 based on the progress of work related to forthcoming PCV internal investigations.

Reduction in radiation dose and mitigation of contamination

Effective dose-reduction at site boundaries and purification of port water to mitigate the impact of radiation on the external environment

- Status of groundwater and seawater on the east side of Turbine Building Units 1-4
- In the Unit 1 intake north side area, the H-3 concentration was below the legal discharge limit of 60,000 Bq/L at all observation holes and remained constant or has been declining overall. The concentration of gross-β radioactive materials increased temporarily from April 2020. It has been increasing or declining at No. 0-3-2 but remains constant or is declining overall.
- In the area between the Unit 1 and 2 intakes, the H-3 concentration has remained below the legal discharge limit of 60,000 Bq/L at all observation holes. It has been increasing or declining at No. 1-14 but has remained constant or been declining at many observation holes overall. The concentration of gross-β radioactive materials has remained constant or been declining at many observation holes overall.
- In the area between the Unit 2 and 3 intakes, the H-3 concentration has been below the legal discharge limit of 60,000 Bq/L at all observation holes and remained almost constant or been declining. The concentration of gross-β radioactive materials has remained almost constant or been declining at many observation holes overall.
- In the area between the Unit 3 and 4 intakes, the H-3 concentration has been below the legal discharge limit of 60,000 Bq/L at all observation holes and remained constant or been declining though increasing and declining at No. 3-3. The concentration of gross-β radioactive materials has also remained constant or been declining overall.
- The concentration of radioactive materials in drainage channels has remained constant overall, despite increasing during rainfall.
- In the open channel area of seawater intake for Units 1 to 4, the concentration of radionuclides in seawater has remained below the legal discharge limit, despite increases in Cs-137 and Sr-90 noted during rainfall. They have also been declining following the completed installation and the connection of steel pipe sheet piles for the sea-side impermeable walls. The concentration of Cs-137 has remained slightly higher in front of the south side impermeable walls and slightly lower on the north side of the east breakwater since March 20, 2019, when the silt fence was transferred to the center of the open channel due to mega float-related construction.
- In the port area, the concentration of radionuclides in seawater has remained below the legal discharge limit, despite
 increases in Cs-137 and Sr-90 observed during rainfall. They have remained below the level of those in the Units 1-4
 intake open channel area and been declining following the completed installation and connection of steel pipe sheet
 piles for the sea-side impermeable walls.
- In the area outside the port, regarding the concentration of radioactive materials in seawater, those of Cs-137 and Sr-90 declined and remained low after steel pipe sheet piles for the sea-side impermeable walls were installed and connected. A temporary increase is sometimes observed due to the influence of weather and marine meteorology.



<Unit 1 intake north side, between Unit 1 and 2 intakes>

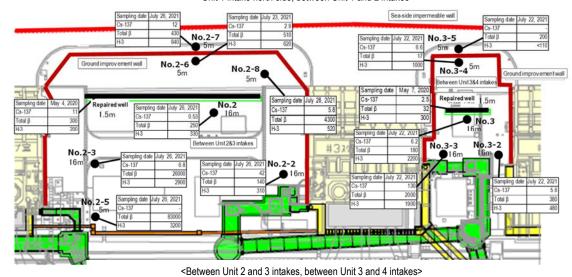


Figure 5: Groundwater concentration on the Turbine Building east side

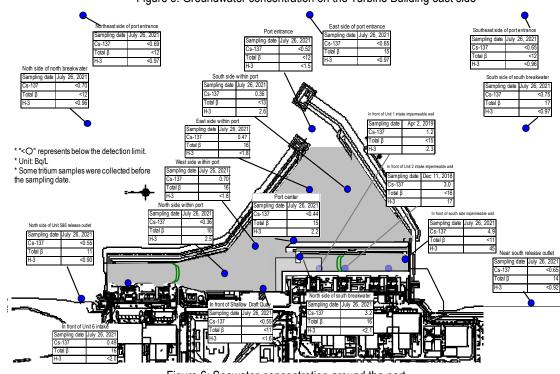


Figure 6: Seawater concentration around the port

Outlook of the number of staff required and efforts to improve the labor environment and conditions

Adequate number of staff will be secured in the long-term, while firmly implementing radiation control of workers. The work environment and labor conditions will be continuously improved by responding to the needs on the site.

> Staff management

- The monthly average total of personnel registered for at least one day per month to work on site during the past quarter from March to May 2021 was approx. 8,600 (cooperating company workers and TEPCO HD employees), which exceeded the monthly average workforce (approx. 6,200). Accordingly, sufficient personnel are registered to work on site.
- It was confirmed with the prime contractors that the estimated manpower necessary for the work in August 2021 (approx. 3,800 workers per day: cooperating company workers and TEPCO HD employees) would be secured at present. The average numbers of workers per day for each month (actual values) of recent 2 years were maintained, with approx. 3,000 to 4,200 (see Figure 7).
- The number of workers from within Fukushima Prefecture slightly increased, as did the number of those from outside. The local employment ratio (cooperating company workers and TEPCO HD employees) as of July 2021 also slightly increased at around 70%.
- The monthly average exposure doses of workers remained at approx. 0.20, 0.21 and 0.22 mSv/month during FY2018, 2019 and 2020*, respectively. *Provisional value for FY2020
- For most workers, the exposure dose was sufficiently within the limit and allowed them to continue engaging in radiation work.

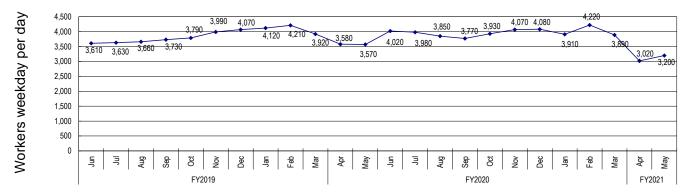


Figure 7: Changes in the average number of workers weekday per day for each month of recent 2 years (actual values)

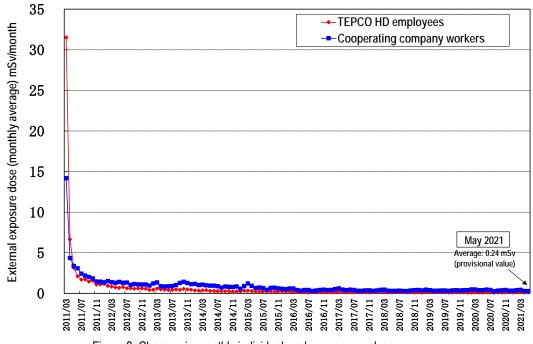


Figure 8: Changes in monthly individual worker exposure dose (monthly average exposure dose since March 2011)

8/9

> Expansion of area without wearing DS2 mask in the Fukushima Daiichi NPS

- Within the work in G zone* outside the Unit 1-4 surrounding protected zone (excluding the inside of Unit 5 and 6), work not involving handling of contaminated facilities or equipment, light work free of concerns about dust stirred up, or moves by vehicles to/from the equipment exchange building or rest house, the requirement to wear a dustproof mask will be waived to reduce the burden on workers. * Gzone: general clothing area
- This revision will be applied from August 2.

➤ COVID-19 infectious disease countermeasures

- As of 15:00, July 28, 2021, 32 TEPCO HD employees and cooperating company workers (including 4 TEPCO HD
 employees) of the Fukushima Daiichi Nuclear Power Station (NPS) had contracted COVID-19. No significant influence
 on decommissioning work, such as a corresponding delay to the work processes due to this infection, had been
 identified.
- Countermeasures have continued to prevent the COVID-19 infection spreading, such as requiring employees to take their temperature before coming to the office, wear masks at all times and avoid the "Three Cs" (Closed spaces, Crowded places, Close-contact settings) by using the rest house in shifts, eat silently and voluntarily refrain from travel to other prefectures and participation in meetings.
- As of July 27, 2021, by the workplace vaccination of COVID-19, 318 TEPCO HD employees and 781 cooperating company workers had received their first dose and 3 and 30, respectively, their second dose.

> Health management of workers in the Fukushima Daiichi NPS

- As health management measures in line with the guidelines of the Ministry of Health, Labour and Welfare (issued in August 2015), a scheme was established and operated, whereby prime contractors confirmed reexamination at medical institutions and the subsequent status of workers who were diagnosed as requiring "detailed examination and treatment" in the health checkup, with TEPCO confirming the operation status by the prime contractors.
- The recent report on the management status of the health checkup during the fourth quarter (January March) in FY2020 confirmed that the prime contractors had provided appropriate guidance and managed operations properly under the scheme. The report on the follow-up status during the third quarter in FY2020 and before confirmed that responses to workers, which had not been completed by the time of the previous report, were being provided on an ongoing basis and checking of operations will continue.

Status of heat stroke cases

- Measures to further prevent heat stroke commenced from April 2021 to cope with the hottest season.
- In FY2021, two workers suffered heat stroke due to work up until July 26 (in FY2020, five workers up until the end of July). Continued measures will be taken to prevent heat stroke.

Status of seawater monitoring within the port (comparison between the highest values in 2013 and the latest values)

"The highest value" -- "the latest value (sampled during July 19-26)"; unit (Bq/L); ND represents a value below the detection limit Note: The Total β measurement values include natural potassium 40 (approx. 12 Bq/L). Summary of TEPCO data as of July 27, 2021 They also include the contribution of yttrium 90, which radioactively balance strontium 90. ND(0.54) Below 1/8 Cesium-134: Cesium-134 : 3.3 (H25/12/24) ND(0.37) ND(0.44) ND(0.52) Below 1/10 Cesium-137: Cesium-137 : 7.3 $(H25/10/11) \rightarrow$ Total β 15 (H25/8/19) ND(12) Below 1/5 : 69 Toritium 2.2 Toritium : 68 (H25/8/19)ND(1.5) **Below 1/40** Cesium-134 : 3.3 (H25/10/17) → ND(0.25) Below 1/10 Cesium-134 : 3.5 (H25/10/17) → ND(0.21) Below 1/10 0.47 Below 1/10 Cesium-137 : 7.8 (H25/10/17) → 0.36 Below 1/20 Cesium-137 : 9 (H25/10/17) → Total β (H25/8/19) → 16 Below 1/4 Total β : 79 (H25/8/19) → ND(13) Below 1/6 Toritium : 67 (H25/8/19) → ND(1.8) Below 1/30 Toritium : 60 (H25/8/19) Below 1/20 Cesium-134 : 4.4 (H25/12/24) → ND(0.30) Below 1/10 Cesium-134 : 32 (H25/10/11) → Below 1/50 ND(0.60) Cesium-137 : 10 (H25/12/24) → 0.70 Below 1/10 Cesium-137 : 73 $(H25/10/11) \rightarrow$ 3.2 Below 1/20 Total β : 60 (H25/7/4)16 Below 1/3 : 320 (H25/8/12) → 16 Below 1/20 Toritium : 59 (H25/8/19) ND(1.8) Below 1/30 : 510 (H25/9/2) ND(2.1) Below 1/200 Toritium Cesium-134 : 5 (H25/12/2)ND(0.31) Below 1/10 Cesium-134: ND(0.46) [East side in the port] [South side in the port] Cesium-137 : 8.4 (H25/12/2) ND(0.36) **Below 1/20** Cesium-137: 4.9 Total B : 69 (H25/8/19) 16 Below 1/4 Total B ND(11) [Port center] 2.5 Toritium : 52 (H25/8/19) → Below 1/20 Toritium 45 [West side in the port] %1: Monitoring commenced in or after March 2014. Monitoring inside the sea-side impermeable walls was [North side in the port] In front of *2: For the point, monitoring was finished from December 12, 2018 due to preparatory work for transfer of mega float. shallow draft *3: For the point, monitoring point was moved from February 6, 2019 due to preparatory work for transfer of mega float. quay] ※4: For the point, monitoring was finished from April 3, 2019 due to preparatory work for transfer of mega float Sea side impermeable wall Cesium-134 60 10 Silt fence for construction 90 10 Cesium-137 (strongly correlate with 30 10 Total β) 60,000 10,000 ND(0.50) 1/5以下 Cesium-134 : 2.8 (H25/12/2) 5.3 (H25/8/5) ND(0.53) 1/10以下 1/10以下 ND(0.55) 1/10以下 Cesium-137 : 5.8 (H25/12/2) 0.49 Cesium-137 : 8.6 (H25/8/5) Source: TEPCO website Analysis results on nuclides of radioactive materials around Fukushima Daiichi 1/2以下 ND(11) 1/3以下 Total B : 46 (H25/8/19) 16 Total B : 40 (H25/7/3)Nuclear Power Station http://www.tepco.co.jp/decommision/planaction/monitoring/index-j.html 1/200以下 Toritium : 24 (H25/8/19) ND(2.0) 1/10以下 : 340 (H25/6/26) → ND(1.6)

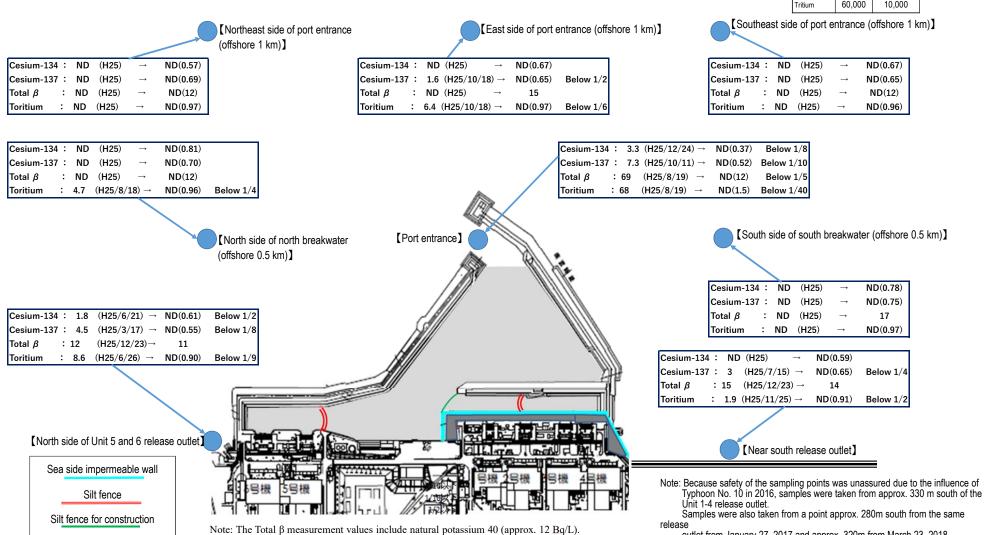
Status of seawater monitoring around outside of the port (comparison between the highest values in 2013 and the latest values)

Unit (Bg/L); ND represents a value below the detection limit; values in () represent the detection limit; ND (2013) represents ND throughout 2013

(The latest values sampled during July 19-26)

	Legal discharge limit	WHO Guidelines for Drinking Water Quality
Cesium-134	60	10
Cesium-137	90	10
Strontium-90 (strongly correlate with Total β)	30	10
Tritium	60,000	10,000

Summary of TEPCO data as of July 27, 2021

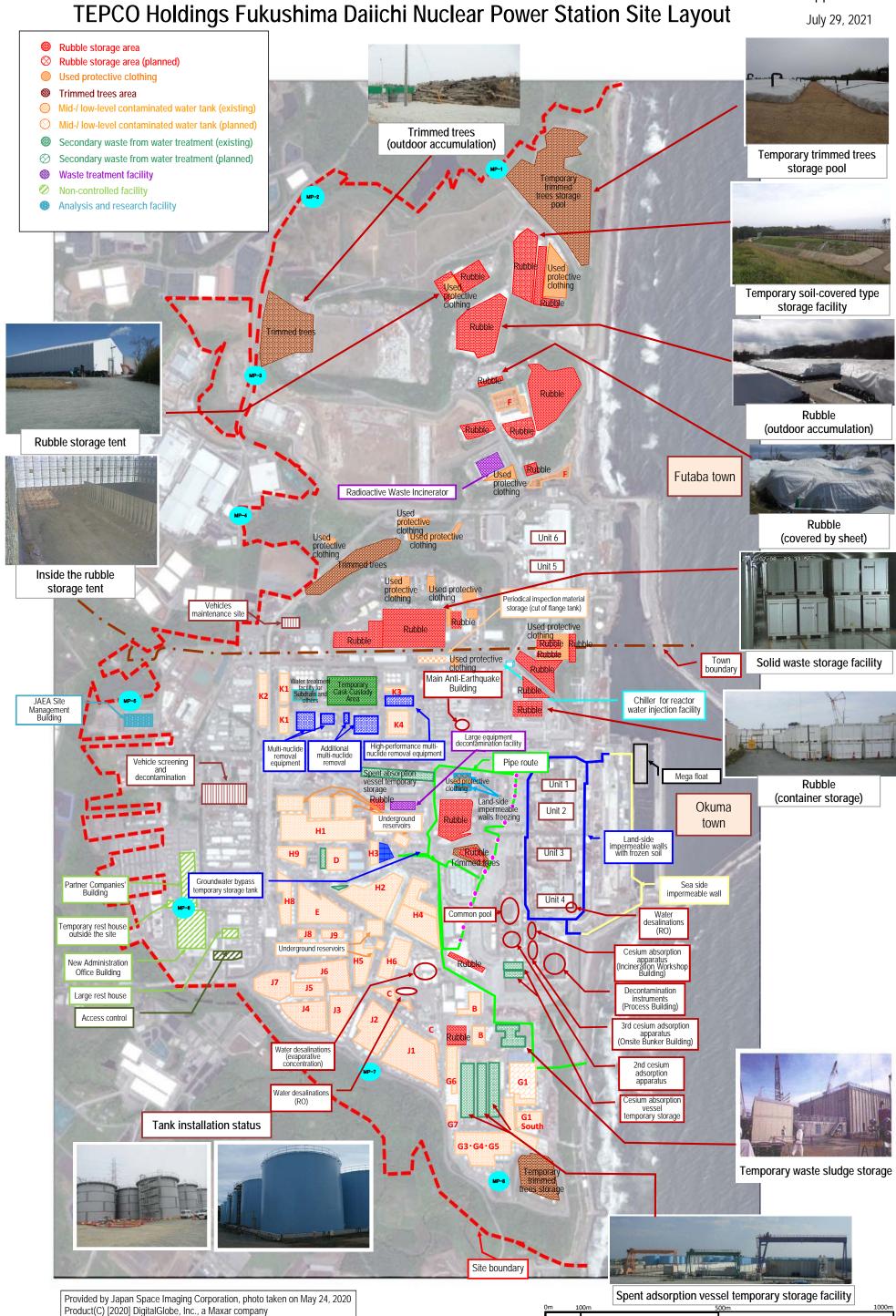


They also include the contribution of yttrium 90, which radioactively balance strontium 90.

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Samples were also taken from a point approx. 280m south from the same

outlet from January 27, 2017 and approx. 320m from March 23, 2018. Source: TEPCO website, Analysis results on nuclides of radioactive materials around Fukushima Daiichi Nuclear Power Station http://www.tepco.co.jp/decommision/planaction/monitoring/index-j.html



Progress toward decommissioning: Fuel removal from the spent fuel pool (SFP)

July 29, 2021 Secretariat of the Team for Countermeasures for

Decommissioning and Contaminated Water Treatment

1/6

Immediate target

Commence fuel removal from the Unit 1-2 Spent Fuel Pools

Unit 1

Toward fuel removal from the Unit 1 spent fuel pool, investigations have been implemented to ascertain the conditions of the fallen roof on the south side and the contamination of the well plug. Based on the results of these investigations, "the method to initially install a large cover over the Reactor Building and then remove rubble inside the cover" was selected to ensure a safer and more secure removal. Work to install a large cover will start from the first half of FY2021. Work continues to complete installation of a large cover by around FY2023 and start fuel removal from FY2027 to FY2028.

<Reference> Progress to date

Rubble removal on the north side of the operating floor started from January 2018 and has been implemented sequentially. In July and August 2019, the well plug, which was misaligned from its normal position, was investigated and in August and September, the conditions of the overhead crane were checked. Based on the results of these investigations as the removal requires more careful work taking dust scattering into consideration, two methods were examined: installing a cover after rubble removal and initially installing a large cover over the Reactor Building and then removing rubble inside the cover.



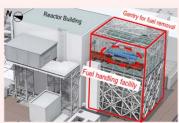
le removal (image)

Fuel removal (image)

Unit 2

Toward fuel removal from the Unit 2 spent fuel pool, based on findings from internal operating floor investigations from November 2018 to February 2019, instead of fully dismantling the upper part of the building, the decision was made to install a small opening on the south side and use a boom crane. Examination continues to start fuel removal from FY2024 to FY2026.

<Reference> Progress to date
Previously, potential to recover the existing
overhead crane and the fuel handling
machine was examined. However, the high
radiation dose inside the operating floor
meant the decision was taken to dismantle
the upper part of the building in November
2015. Findings from internal investigations of
the operating floor from November 2018 to
February 2019 underlined the potential to
conduct limited work there and the means of
accessing from the south side had been
examined.



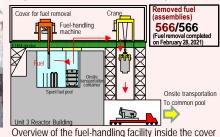
Overview of fuel removal (bird's-eye view)

Unit 3

Prior to the installation of a cover for fuel removal, removal of large rubble from the spent fuel pool was completed in November 2015. To ensure safe and steady fuel removal, training of remote control was conducted at the factory using the actual fuel-handling machine which will be installed on site (February – December 2015). Measures to reduce dose on the Reactor Building top floor (decontamination, shields) were completed in December 2016. Installation of a cover for fuel removal and a fuel-handling machine is underway from January 2017. Installation of the fuel removal cover was completed on February 23, 2018.

Toward fuel removal, the rubble retrieval training inside the pool, which was scheduled in conjunction with fuel removal training, started from March 15, 2019, and started fuel removal from April 15, 2019. Fuel removal was completed on February 28, 2021.







Fuel removal (566th assembly) (February 26, 2021)

Unit 4

In the Mid- and-Long-Term Roadmap, the target of Phase 1 involved commencing fuel removal from inside the spent fuel pool (SFP) of the 1st Unit within two years of completion of Step 2 (by December 2013). On November 18, 2013, fuel removal from Unit 4, or the 1st Unit, commenced and Phase 2 of the roadmap started.

On November 5, 2014, within a year of commencing work to fuel removal, all 1,331 spent fuel assemblies in the pool had been transferred. The transfer of the



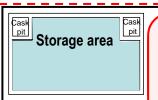
Fuel removal status

remaining non-irradiated fuel assemblies to the Unit 6 SFP was completed in December 22, 2014. (2 of the non-irradiated fuel assemblies were removed in advance in July 2012 for fuel checks)

This marks the completion of fuel removal from the Unit 4 Reactor Building. Based on this experience, fuel assemblies will be removed from other Unit pools.

* A part of the photo is corrected because it includes sensitive information related to physical protection.

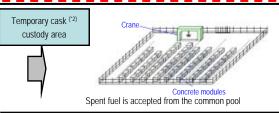
Common pool



An open space will be maintained in the common pool (Transfer to the temporary cask custody area)

Progress to date

- The common pool has been restored to a condition allowing it to re-accommodate fuel to be handled (November 2012)
- Loading of spent fuel stored in the common pool to dry casks commenced (June 2013)
- Fuel removal from the Unit 4 spent fuel pool began to be received (November 2013 - November 2014)
- Fuel removal from the Unit 3 spent fuel pool began to be received (April 2019 February 2021)



Operation commenced on April 12, 2013; from the cask-storage building, transfer of 9 existing dry casks completed (May 21, 2013); fuel stored in the common pool sequentially transferred.

<Glossa

- (*1) Operating floor: During regular inspection, the roof over the reactor is opened while on the operating floor, fuel inside the core is replaced and the core internals are inspected.
- (*2) Cask: Transportation container for samples and equipment, including radioactive materials.

Immediate target

Identify the plant status and commence R&D and decontamination toward fuel debris retrieval

Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment

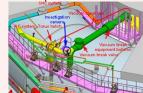
Investigation into TIP Room of the Unit 1 Reactor Building

- To improve the environment for future investigations inside the PCV, etc., an investigation was conducted from September 24 to October 2, 2015 at the TIP Room(*1). (Due to high dose around the entrance in to the TIP Room, the investigation of dose rate and contamination distribution was conducted through a hole drilled from the walkway of the Turbine Building,
- The investigative results identified high dose at X-31 to 33 penetrations^(*2) (instrumentation penetration) and low dose at
- As it was confirmed that work inside the TIP room would be available, the next step will include identification of obstacles which will interfere the work inside the TIP Room and formulation of a plan for dose reduction

Investigation in the leak point detected in the upper part of the Unit 1 Suppression Chamber (S/C(*3)) Investigation in the leak point detected in the upper part of Unit 1 S/C from May 27, 2014 from one

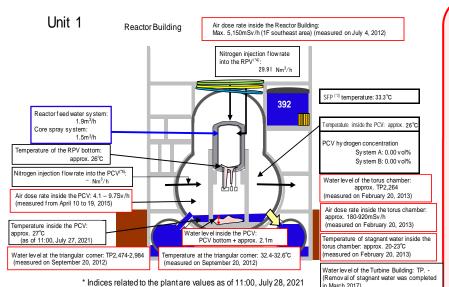
expansion joint cover among the lines installed there. As no leakage was identified from other parts, specific methods will be examined to halt the flow of water and repair the PCV.





Leak point

Image of the S/C upper part investigation



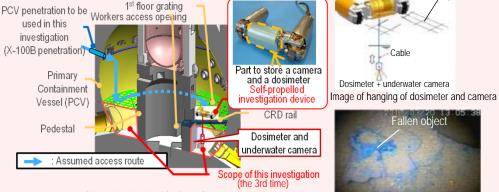
	1st (Oct 2012)	- Acquiring images - Measuring air temperature and dose rate - Measuring water level and temperature - Sampling stagnant water - Installing permanent monitoring instrumentation
Investigations inside PCV	2nd (Apr 2015)	Confirming the status of PCV 1st floor - Acquiring images - Measuring air temperature and dose rate - Replacing permanent monitoring instrumentation
3 rd	3 rd (Mar 2017)	Confirming the status of PCV 1st basement floor - Acquiring images - Measuring and dose rate - Sampling deposit - Replacing permanent monitoring instrumentation
Leakage points from PCV	- PCV vent pipe vacuum break line bellows (identified in May 2014) - Sand cushion drain line (identified in November 2013)	

Status of investigation inside the PCV

Prior to fuel debris retrieval, an investigation inside the PCV will be conducted to inspect the status there including the location of fuel debris.

[Investigative outline]

- In April 2015, a device, which entered the inside of the PCV through a narrow access opening (bore: φ 100 mm), collected information such as images and airborne dose inside the PCV 1st floor.
- In March 2017, the investigation using a self-propelled investigation device, conducted to inspect the spreading of debris to the basement floor outside the pedestal, took images of the PCV bottom status for the first time. The status inside the PCV will continue to be examined based on the collected image and dose data.



<Image of investigation inside the PCV>

Image near the bottom

Fallen object

Cable

Grating

Capturing the location of fuel debris inside the reactor by measurement using muons

Period	Evaluation results
Feb - May 2015	Confirmed that there was no large fuel in the reactor core.

<Glossarv:

- (*1) TIP (Traversing In-core Probe)
- (*2) Penetration: Through-hole of the PCV
- (*3) S/C (Suppression Chamber): Suppression pool, used as the water source for the emergent core cooling system.
- (*4) SEP (Spent Fuel Pool):
- (*5) RPV (Reactor Pressure Vessel)
- (*6) PCV (Primary Containment Vessel)

July 29, 2021

Immediate target

Identify the plant status and commence R&D and decontamination toward fuel debris retrieval

in December 2020)

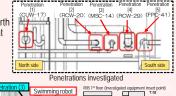
Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment

Installation of an RPV thermometer and permanent PCV supervisory instrumentation

- (1) Replacement of the RPV thermometer
- As the thermometer installed at the Unit 2 RPV bottom after the earthquake had broken in February 2014, it was excluded from the monitoring thermometers.
- In April 2014, removal of the broken thermometer failed and was suspended. Rust-stripping chemicals were injected and the broken thermometer was removed in January 2015. A new thermometer was reinstalled in March. The thermometer has been used as a part of permanent supervisory instrumentation since April.
- (2) Reinstallation of the PCV thermometer and water-level gauge
- Some of the permanent supervisory instrumentation for PCV could not be installed in the planned locations due to interference with existing grating (August 2013). The instrumentation was removed in May 2014 and new instruments were reinstalled in June 2014. The trend of added instrumentation will be monitored for approx. one month to evaluate its validity.
- The measurement during the installation confirmed that the water level inside the PCV was approx. 300mm from the bottom.

Investigative results on torus chamber walls

- July 2014, the torus chamber walls were investigated (on the north the east-side walls) using equipment specially developed for that purpose (a swimming robot and a floor traveling robot).
- At the east-side wall pipe penetrations (five points), "the status" and "existence of flow" were checked.
- A demonstration using the above two types of underwater wall investigative equipment showed how the equipment could check the status of penetration.
- · Regarding Penetrations [1]-[5], the results of checking the sprayed tracer (*5) by camera showed no flow around the penetrations. (investigation by the swimming robot)
- Regarding Penetration 3, a sonar check showed no flow around the penetrations. (investigation by the floor traveling robot) Floor traveling robot



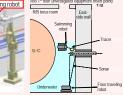
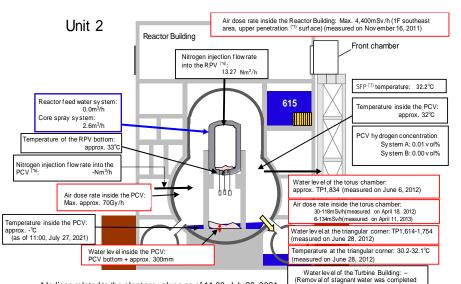


Image of the torus chamber east-side cross-sectional investigation



Investigations inside PCV	1st (Jan 2012)	- Acquiring images - Measuring air temperature
	2nd (Mar 2012)	- Confirming water surface - Measuring water temperature - Measuring dose rate
	3rd (Feb 2013 – Jun 2014)	- Acquiring images - Sampling stagnant water - Measuring water level - Installing permanent monitoring instrumentation
	4th (Jan – Feb 2017)	- Acquiring images - Measuring dose rate - Measuring air temperature
	5th (Jan 2018)	- Acquiring images - Measuring dose rate - Measuring air temperature
	6th (Feb 2019)	- Acquiring images - Measuring dose rate - Measuring air temperature - Grasping characteristics of a portion of deposit
Leakage points from PCV	- No leakage from torus chamber rooftop - No leakage from all inside/outside surfaces of S/C	

* Indices related to the plant are values as of 11:00, July 28, 2021

Status of investigation inside the PCV

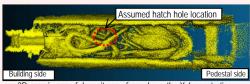
Prior to fuel debris retrieval, an investigation inside the PCV will be conducted to inspect the status there including the location of fuel debris. [Investigative outline]

- Investigative devices such as a robot will be injected from Unit 2 X-6 penetration⁽¹⁾ and access the inside of the pedestal using the CRD rail. [Progress status]
- On January 26 and 30, 2017, a camera was inserted from the PCV penetration to inspect the status of the CRD replacement rail on which the robot will travel. On February 9, deposit on the access route of the self-propelled investigative device was removed and on February 16, the inside of the PCV was investigated using the device.

 The results of this series of investigations confirmed fallen and deformed gratings and a quantity of deposit inside the pedestal.
- On January 19, 2018, the status below the platform inside the pedestal was investigated using an investigative device with a hanging mechanism. From the analytical results of images obtained in the investigation, deposits probably including fuel debris were found at the bottom of the pedestal. In addition, multiple parts higher than the surrounding deposits were also detected. We presumed that there were multiple routes of fuel debris falling. Obtained data were processed in panoramic image visualization to acquire clearer
- · On February 13, 2019, an investigation touching the deposits at the bottom of the pedestal and on the platform was conducted and confirmed that the pebble-shaped deposits, etc. could be moved and that hard rock-like deposits that could not be gripped may exist.
- In addition, images, etc. would help determine the contour and size of the deposits could be collected by moving the investigative unit closer to the deposits than the previous investigation.
- On October 28, 2020, as a preparatory stage of the PCV internal investigation and the trial retrieval, a contact investigation into deposits inside the penetration (X-6 penetration) was conducted. In this investigation, a guide pipe incorporating an investigative unit inserted into the penetration. By the contact, it was confirmed that deposits inside the penetration did not deformed and unstuck.
- On October 30, 2020, a 3D scan investigation was conducted, measuring deposits by the 3D scan sensor mounted on the top of the

Information obtained in the investigation will be utilized in the mockup test of the equipment to remove deposits inside the X-6 penetration.







<3D scan image of deposit seen from above the X-6 penetration>

<Work in front of the penetration>

Capturing the location of fuel debris inside the reactor by measurement using muons

Period	Evaluation results
Mar – Jul 2016	Confirmed the existence of high-density materials, which was considered as fuel debris, at the bottom of RPV, and in the lower part and the outer periphery of the reactor core. It was assumed that a large part of fuel debris existed at the bottom of RPV.

1) Penetration: Through-hole of the PCV *2) SFP (Spent Fuel Pool) (*3) RPV (Reactor Pressure Vessel) (*4) PCV (Primary Containment Vessel) (*5) Tracer: Material used to trace the fluid flow. Clay particles

Immediate target

Identify the plant status and commence R&D and decontamination toward fuel debris retrieval

Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment

Water flow was detected from the Main Steam Isolation Valve* room

On January 18, 2014, a flow of water from around the door of the Steam Isolation Valve room in the Reactor Building Unit 3 1st floor northeast area to the nearby floor drain funnel (drain outlet) was detected. As the drain outlet connects with the underground part of the Reactor Building, there is no possibility of outflow from the building.

From April 23, 2014, image data has been acquired by camera and the radiation dose measured via pipes for measurement instrumentation, which connect the air-conditioning room on the Reactor Building 2nd floor with the Main Steam Isolation Valve Room on the 1st floor. On May 15, 2014, water flow from the expansion joint of one Main Steam Line was detected.

This is the first leak from PCV detected in the Unit 3. Based on the images collected in this investigation, the leak volume will be estimated and the need for additional investigations will be examined. The investigative results will also be utilized to examine water stoppage and PCV repair methods.

* Main Steam Isolation Valve: A valve to shut off the steam generated from the Reactor in an emergency

Unit 3 Air dose rate inside the Reactor Building: Max. 4,780mSv/h (1E northeast area in front of the equipment hatch) (measured on November 27, 2012) Fuel-handling machine Crane Dome roof FHM girder Nitrogen injection flow rate Removed fuel (assemblies) Reactor feed water system: 1.4m3/l Shield 566/566 16.70 Nm³/ Core spray system: 1.5m3 (Fuel removal completed on February 28, 2021) SFP (*3) temperature: 28.4°C Temperature of the RPV bottom approx. 28°C Temperature inside the PCV: approx. 29°C Air dose rate inside the PCV (*5): PCV by drogen concentration Max approx 1Sv/h (measured on October 20, 2015) System A: 0.04 vol% System B: 0.04 vol% Water level of the torus chamber; approx. TP1.934 (measured on June 6, 2012) Temperature inside the PCV Air dose rate inside the torus chamber: 100-360mSv/ easured on July 11, 2012) (as of 11:00, July 27, 2021) Water lev el at the triangular corner: TP1,714 (measured on June 6, 2012) Water lev el inside the PCV: PCV bottom + approx. 6.3m Water level of the Turbine Building: (measured on October 20, 2015) (Removal of stagnant water was

* Indices related to the plant are values as of 11:00, July 28, 2021

Investigations inside PCV	1st (Oct – Dec 2015)	Acquiring images - Measuring air temperature and dose rate Measuring water level and temperature - Sampling stagnant water Installing permanent monitoring instrumentation (December 2015)
	2nd (Jul 2017)	- Acquiring images - Installing permanent monitoring instrumentation (August 2017)
Leakage points from PCV	- Main steam pipe bellows (identified in May 2014)	

Investigative results into the Unit 3 PCV equipment hatch using a small investigation device

As part of the investigation into the PCV to facilitate fuel debris retrieval, the status around the Unit 3 PCV
equipment hatch was investigated using a
small self-traveling investigation device on

November 26, 2015.

Given blots such as rust identified below the water level inside the PCV, there may be a leakage from the seal to the

extent of bleeding.

Methods to investigate and repair the parts, including other PCV penetrations with a similar structure, will be considered.



Investigation inside the PCV

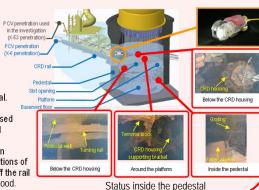
Prior to fuel debris retrieval, the inside of the Primary Containment Vessel (PCV) was investigated to identify the status there including the location of the fuel debris.

[Investigative outline

- The status of X-53 penetration⁽¹⁴⁾, which may be under the water and which is scheduled for use to investigate the inside of the PCV, was investigated using remote-controlled ultrasonic test equipment. The results showed that the penetration was not under the water (October 22-24, 2014).
- For the purpose of confirming the status inside the PCV, an investigation device was inserted into the PCV from X-53 penetration on October 20 and 22, 2015 to obtain images, data of dose and temperature and sample stagnant water. No damage was identified on the structure and walls inside the PCV and the water level was almost identical with the estimated value. In addition, the dose inside the PCV was confirmed to be lower than in other Units.

 PCV penetration

 (KS) penetration
- In July 2017, the inside of the PCV was investigated using the underwater ROV (remotely operated underwater vehicle) to inspect the inside of the pedestal.
- Analysis of image data obtained in the investigation identified damage to multiple structures and the supposed core internals. Consideration about fuel removal based on the obtained information will continue.
- Videos obtained in the investigation were reproduced in 3D. Based on the reproduced images, the relative positions of the structures, such as the rotating platform slipping off the rail with a portion buried in deposits, were visually understood.



Capturing the location of fuel debris inside the reactor by measurement using muons

Period	Evaluation results
May - Sep 2017	The evaluation confirmed that no large lump existed in the core area where fuel had been placed and that part of the fuel debris potentially existed at the bottom of the RPV.

<Glossary>

(*3) SFP (Spent Fuel Pool) (*4) RPV (Reactor Pressure Vessel) (*5) PCV (Primary Containment Vessel) (*6) Penetration: Through-hole of the PCV

Immediate target

Lower permeable layer

(5) Land-side impermeable wall

Low-permeable layer

Stably continue reactor cooling and stagnant water treatment, and improve reliability

Work to improve the reliability of the circulation water injection cooling system and pipes to transfer stagnant water.

- Operation of the reactor water injection system using Unit 3 Condensate Storage Tank (CST) as a water source commenced (from July 5, 2013). Compared to the previous systems the reliability of the reactor water injection system was enhanced, e.g. by increasing the amount of water-source storage and enhancing durability.

 To reduce the risk of contaminated-water leakage, the circulation loop was shortened by installing a reverse osmosis (RO) device in the Unit 4 Turbine Building within the circulation.
- loop, comprising the transfer of contaminated water, water treatment and injection into the reactors. Operation of the installed RO device started from October 7 and 24-hour operation started from October 20. Installation of the new RO device inside the building shortened the circulation loop from approx. 3 to 0.8 km.
- To accelerate efforts to reduce the radiation density in stagnant water inside the buildings, circulating purification of stagnant water inside the buildings stared on the Unit 3 and 4 side on February 22 and on the Unit 1 and 2 side on April 11.
- For circulating purification, a new pipe (stagnant water purification line) divided from the water treatment equipment outlet line was installed to transfer water purified at the water treatment equipment to the Unit 1 Reactor Building and the Unit 2-4 Turbine Buildings.

 * The entire length of contaminated water transfer.

 * The entire length of contaminated water transfer. * The entire length of contaminated water transfer
- The risks of stagnant water inside the buildings will continue to be reduced in addition to reduction of its storage

pipes is approx. 2.1km, including the transfer line of surplus water to the upper heights (approx. : Existing line : RO line inside : Stagnant water the building purification line RO-treated water Process Main Building / High Temperature Incinerator Condensed water RO Sr reduced water R/B RO SARRY/ KURION Storage SPT quipme removal of Cs and Si tank (B) (existing)

Progress status of dismantling of flange tanks

To facilitate replacement of flanged tanks, dismantling of flanged tanks started in H1 east/H2 areas in May 2015. Dismantling of all flanged tanks was completed in H1 east area (12 tanks) in October 2015, in H2 area (28 tanks) in March 2016, in H4 area (56 tanks) in May 2017, in H3 B area (31 tanks) in September 2017, in H5 and H5 north areas (31 tanks) in June 2018, in G6 area (38 tanks) in July 2018. H6 and H6 north areas (24 tanks) in September 2018 and G4 south area (17 tanks) in March 2019.





Start of dismantling in H1 east area

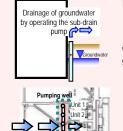
After dismantling in H1 east area

Completion of purification of contaminated water (RO concentrated salt water)

Contaminated water (RO concentrated salt water) is being treated using seven types of equipment including the multi-nuclide removal equipment (ALPS). Treatment of the RO concentrated salt water was completed on May 27, 2015, with the exception of the remaining water at the tank bottom. The remaining water will be treated sequentially toward dismantling the tanks.

The strontium reduced water from other facilities than the multi-nuclide removal equipment will be re-purified in the multi-nuclide removal equipment to further reduce risks.

Preventing groundwater from flowing into the Reactor Buildings



·Length: approx. 1.500m

(Mountain side

Freezing plant

Land-side

mpermeable walls

Reducing groundwater inflow by pumping sub-drain water To reduce groundwater flowing into the buildings, pumping-up of groundwater from wells (subdrains) around the buildings started on September 3, 2015. Pumped-up groundwater was purified at dedicated facilities and released after TEPCO and a third-party organization confirmed that its quality met operational targets.

Via a groundwater bypass, reduce the groundwater level around the Building and groundwater inflow into the Building

Measures to pump up groundwater flowing from the mountain side upstream of the Building to reduce the groundwater inflow (groundwater bypass) have been implemented

The pumped up groundwater is temporarily stored in tanks and released after TEPCO and a thirdparty organization have confirmed that its quality meets operational targets. Through periodical monitoring, pumping of wells and tanks is operated appropriately.

At the observation holes installed at a height equivalent to the buildings, the trend showing a decline in groundwater levels is checked.

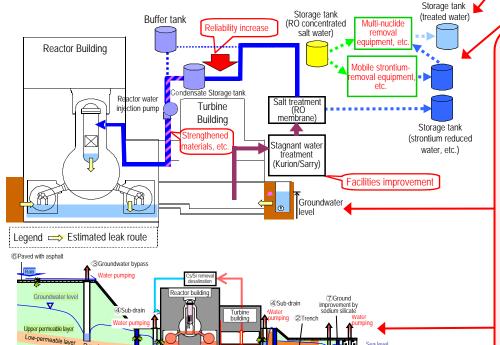
The analytical results on groundwater inflow into the buildings based on existing data showed a declining trend.

Installing land-side impermeable walls with frozen soil around Units 1-4 to prevent the inflow of groundwater into the building



In March 2018, construction of the land-side impermeable walls was completed, except for a portion of the depth, based on a monitoring result showing that the underground temperature had declined below 0°C in almost all areas, while on the mountain side, the difference between the inside and outside increased to approx. 4-5 m. The 21st Committee on Countermeasures for Contaminated Water Treatment, held on March 7, 2018, evaluated that together with the function of sub-drains, etc., a water-level management system to stably control groundwater and isolate the buildings from it had been established and had allowed a significant reduction in the amount of contaminated water generated.

For the unfrozen depth, a supplementary method was implemented and it was confirmed that temperature of the part declined below 0°C by September 2018. From February 2019, maintenance operation started at all sections.



⑤Land-side impermeable wall

Sea-side impermeable wall

Immediate targets

- Reduce the effect of additional release from the entire power station and radiation from radioactive waste (secondary water treatment waste, rubble, etc.) generated after the accident, to limit the effective radiation dose to below 1mSv/year at the site boundaries.
- Prevent contamination expansion in sea, decontamination within the site

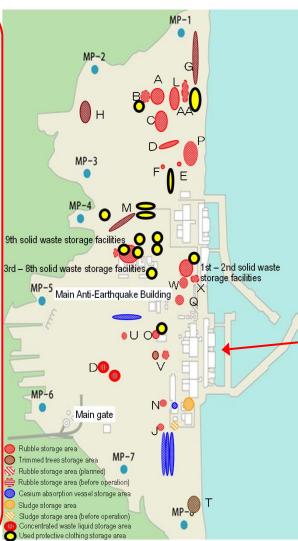
Optimization of radioactive protective equipment Based on the progress of measures to reduce environmental dosage on site, the site is categorized into two zones: highly contaminated area around Unit 1-4 buildings, etc. and other areas to optimize protective equipment according to each category aiming at improving safety and productivity by reducing load during work. From March 2016, limited operation started. From March and September 2017, the G Zone was expanded Rzono [Anorak area] %1 Yzono [Coveral area] %2 Gzono [General war area] %3 Line 1 see size Line 3 see size (size 2 Sm above see cvo) (size 2 Sm above see cvo)



*1 For works in buildings including water-treatment facilities [multi-nuclide removal equipment etc.1 (excluding site visits), wear a full-face mask

Or double coveralls

*2 For works in tank areas containing concentrated salt water or Sr-treated water (excluding works not handling concentrated salt water, etc., patrol, on-site investigation for work planning, and site visits) and works related to tank transfer lines, wear a full-face mask. Specified light works (patrol, monitoring, delivery of goods brought from outside, etc.)



Installation of dose-rate monitors

To help workers in the Fukushima Daiichi Nuclear Power Station precisely understand the conditions of their workplaces, a total of 86 dose-rate monitors were installed by January 4, 2016.

These monitors allow workers to confirm real time on-site dose rates at their workplaces.

Workers are also able to check concentrated data through large-scale displays installed in the Main Anti-Earthquake Building and the access control facility.



Installation of Dose-rate monitor

Installation of sea-side impermeable walls

To prevent the outflow of contaminated water into the sea, sea-side impermeable walls have been installed.

Following the completed installation of steel pipe sheet piles on September 22, 2015, connection of these piles was conducted and connection of sea-side impermeable walls was completed on October 26, 2015. Through these works, closure of sea-side impermeable walls was finished and the contaminated water countermeasures have been greatly advanced.



Installation of steel pipe sheet piles for sea-side impermeable wall

Status of the large rest house

A large rest house for workers was established and its operation commenced on May 31, 2015.

Spaces in the large rest house are also installed for office work and collective worker safety checks as well as taking rest.

On March 1, 2016 a convenience store opened in the large rest house. On April 11, operation of the shower room started. Efforts will continue to improve convenience of workers.

